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# Design and Comparison of Two Advanced Core Control Systems for Flexible Operation of Pressurized Water Reactors

Guillaume Dupré<sup>a,b,\*</sup>, Philippe Chevrel<sup>b</sup>, Mohamed Yagoubi<sup>b</sup>, Alain Grossetête<sup>a</sup>

<sup>a</sup>Reactor Process Department, Framatome, 1 pl. Jean Millier, Paris La Défense, 92400, France

<sup>b</sup>LS2N (UMR 6004), IMT Atlantique, 4 rue Alfred Kastler, Nantes, 44307, France

## Abstract

This paper focuses on the design of advanced core control systems for future generations of pressurized water reactors. The objective is to improve the flexibility of nuclear power plants to cope with the rapid growth of renewable energies. In practice, this means that the average coolant temperature, the axial power distribution of the reactor core and the position of the control rods have to be properly regulated during power variations. In previous work, conducted by the same authors, two promising approaches were investigated: 1) fixed-structure gain-scheduled control and 2) nonlinear model predictive control. Here, both methods are tested according to industry standards in an attempt to determine the best one for our problem. To achieve this, two different controllers are designed using a new multipoint kinetic model of the reactor core, which provides an accurate representation of the axial power distribution. The advantages and drawbacks of both design methodologies are discussed and then compared on PWRSimu, an intermediate complexity pressurized water reactor simulator developed by Framatome.

**Keywords:** Nuclear Power Plants, Gain-Scheduling, Model Predictive Control

## Nomenclature

ACT	Average Coolant Temperature
AO	Axial Offset
GSC	Gain-Scheduled Control
MS	Multiple-Shooting
NMPC	Nonlinear Model Predictive Control
NP	Nominal Power
PWR	Pressurized Water Reactor
SS	Single-Shooting

## 1. Introduction

In order to address climate change, many countries are seeking to replace fossil fuel power plants by renewable energy sources [1]. This energy transition poses new challenges in terms of management of the electrical network. Since excess electricity cannot currently be stored on a large scale, generation and consumption have to be continuously balanced to ensure grid stability. For this purpose, a number of power plants must be flexible, i.e., capable of adjusting their power output on demand. However, unlike fossil fuel power plants, which are easily controllable, part of renewable energy sources, such as wind turbines or solar

panels, are inherently intermittent. It is therefore essential to enhance the flexibility of conventional power plants to prepare for the upcoming expansion of renewables [2].

A nuclear power plant can be operated either in base-load or in load-following mode [3]. In base-load mode, the power output of the plant remains constant regardless of electricity demand. Most nuclear power plants around the world are still operated in base-load mode because it is simpler and more cost-effective. In load-following mode, the power output of the plant is adjusted depending on the forecasted electricity consumption. Typically, the grid operator sends a daily load profile in advance to the plant operator, who will then set the power target of the turbine accordingly. Whether in base-load or in load-following mode, the nuclear power plant can also provide frequency control to the grid. This is achieved by a dedicated controller that automatically adjusts the power output of the turbine to offset small load imbalances within seconds.

Flexible operation of a nuclear power plant is directly related to the design of the core control system [4]. The main task of the core control system is to maintain the average coolant temperature (ACT) and the axial power distribution of the reactor core, or axial offset (AO), within appropriate limits. These limits are defined upstream to ensure acceptable performance levels and safe operation of the plant. During power variations, ACT regulation is achieved by moving several neutron-absorbing control rods inside the reactor core. However, their movements have a detrimental impact on its axial power distribution. Initially, the movements of the control rods were mitigated by

\*Corresponding author

Email addresses: [guillaume.dupre@framatome.com](mailto:guillaume.dupre@framatome.com) (Guillaume Dupré), [philippe.chevrel@imt-atlantique.fr](mailto:philippe.chevrel@imt-atlantique.fr) (Philippe Chevrel), [mohamed.yagoubi@imt-atlantique.fr](mailto:mohamed.yagoubi@imt-atlantique.fr) (Mohamed Yagoubi), [alain.grossetete@framatome.com](mailto:alain.grossetete@framatome.com) (Alain Grossetête)

43 adjusting the boron concentration of the primary coolant.  
44 This strategy was limiting in terms of flexibility because  
45 the boron concentration of the coolant can only change  
46 slowly and gradually decreases over the fuel burn-up cycle.  
47 For example in mode A, the maximum power variation rate  
48 of the plant is typically restricted to 1-2 %NP/min at the  
49 beginning of the cycle, and 0.2 %NP/min at the end. To  
50 overcome these limitations, advanced core control systems  
51 usually comprise two separate groups, or banks, of control  
52 rods [4]. In this way, one bank of control rods can be used  
53 to perform power variations by, e.g., controlling the ACT,  
54 while the other can be used to reduce axial power distribu-  
55 tion disturbances. Thus, flexibility is increased with power  
56 variation rates of up to 5 %NP/min. After power varia-  
57 tions, the xenon distribution of the reactor core will slowly  
58 change for several hours until steady-state is reached. Dur-  
59 ing this time, the evolution of xenon concentration has  
60 to be counterbalanced to prevent the AO of the reactor  
61 core from drifting away. In highly advanced core control  
62 systems, such as mode T [5], this can be done either by  
63 moving the control rods or by adjusting the boron concen-  
64 tration of the primary coolant. The main motivation for  
65 using the control rods is to minimize the volume of effluent.  
66 Yet, this can decrease the maneuvering capabilities of the  
67 plant, as the control rods may not be properly positioned  
68 for a quick return to nominal power. Maximum flexibility  
69 is achieved when xenon poisoning is compensated by boron  
70 concentration adjustments and when the level of insertion  
71 of the control rods is controlled.

72 In practice, industrial core control systems manage to  
73 meet all the aforementioned requirements using single in-  
74 put single output Proportional-Integral-Derivative control  
75 along with sophisticated logic rules. Even if this approach  
76 gives satisfactory results for now, it may become insuf-  
77 ficient against more stringent grid management criteria.  
78 Fortunately, a wide variety of core control systems have  
79 already been proposed (see [6] and references therein).  
80 Popular [methods](#) include LQG/LTR control,  $H_\infty$  control,  
81 sliding-mode control, model predictive control and gain-  
82 scheduled control. Nevertheless, these conceptual core  
83 control systems do not [tackle](#) all the [issues](#) that arise from  
84 the flexible operation of nuclear power plants. Hence,  
85 this paper proposes to bridge the gap between industry  
86 and academia by developing two different advanced core  
87 control systems using modern design techniques. In view  
88 of the strong operational constraints that are placed on  
89 the nuclear industry, two promising approaches have been  
90 identified: [fixed-structure gain-scheduled control](#) [7, 8] and  
91 [nonlinear model predictive control](#) [9].

92 To the best of our knowledge, very few papers have tried  
93 to apply these techniques to the control of a nuclear reac-  
94 tor core. For instance, it seems that [7, 8] are the first  
95 ones that studied [fixed-structure gain-scheduled control](#) in  
96 this context. Regarding [model predictive control](#), only a  
97 handful of examples have been reported in the scientific  
98 literature. In [10, 11], a [linear model predictive controller](#)  
99 is designed to achieve both ACT and AO regulation of a

100 reactor core that is operated in load-following mode. How-  
101 ever, the boron concentration of the coolant is adjusted by  
102 another module and the nuclear power plant does not pro-  
103 vide frequency control to the grid. In another paper [12],  
104 the power output of a reactor core is controlled by a robust  
105 nonlinear model predictive controller. Yet, this theoretic-  
106 al work is not industrially relevant as it does not even  
107 address AO regulation. By contrast, the main contribu-  
108 tion of this paper is to conduct a realistic and compre-  
109 hensive study on the design of nuclear reactor core con-  
110 trol systems. The objective is to give practical insights  
111 and methodological guidelines that could help industrial  
112 practitioners working in nuclear engineering. Two mod-  
113 ern control techniques, which were seldom tested in this  
114 field, are compared against the technical specifications of  
115 Framatome. In order to ensure consistency with industry  
116 standards, the actuators and control objectives are identi-  
117 cal to those considered in mode T [5].

118 Following on from the work carried out in [8], the first  
119 core control system is designed based on a fixed-structure  
120  $H_2/H_\infty$  gain-scheduled control approach. The main im-  
121 provement is that the controller is tuned using a multipoint  
122 kinetic model of the reactor core rather than a point ki-  
123 netic one. In addition, the mathematical formulation of  
124 the  $H_2/H_\infty$  synthesis problem has been refined and made  
125 more readable. Moreover, the gains are now simultane-  
126 ously tuned at each operating point and smoothly inter-  
127 polated as a function of the scheduling variable [13]. This  
128 greatly simplifies the design of the controller, as it avoids  
129 *a posteriori* interpolation, or post-processing, of the gains.  
130 The second core control system is designed based on a  
131 nonlinear model predictive control approach [14, 15]. This  
132 time, the multipoint kinetic model of the reactor core is  
133 embedded into the controller to predict the future response  
134 of the plant. Thus, the control inputs can be calculated  
135 online by repeatedly solving a constrained finite horizon  
136 optimal control problem. To this end, the model of the  
137 reactor core is reduced using singular perturbation theory  
138 so that it can be efficiently simulated by the controller.

139 The remainder of this paper is organized as follows:  
140 Section II describes the system under study and formal-  
141 izes the control objectives; Section III presents the design  
142 methodology of the fixed-structure  $H_2/H_\infty$  gain-scheduled  
143 controller and Section IV presents the design methodol-  
144 ogy of the nonlinear model predictive controller; Then,  
145 the advantages and drawbacks of both techniques are dis-  
146 cussed in Section V based on a typical load-following  
147 MATLAB<sup>®</sup>/Simulink<sup>®</sup> simulation scenario; Finally, con-  
148 clusions and perspectives are drawn in Section VI.

## 149 2. Problem Formulation

### 150 2.1. System Modeling

151 To design the core control systems, a new multipoint  
152 core kinetic model of a pressurized water reactor (PWR),  
153 drawn in Fig. 1, has been developed (a brief overview

of the model is given in [16] Appendix F). Its main advantage, compared to the former point kinetic model presented in [7, 16], is that the behavior of the AO is more accurately represented since the reactor core is now divided into six meshes. However, increasing the number of meshes also increases the size of the model. As illustrated in Fig. 1, every new mesh requires five additional states: two average neutron  $n$  and delayed neutron  $c$  densities, two iodine  $I$  and xenon  $X_e$  concentrations, and one inlet temperature  $T$ . For each mesh, neutron dynamics are described by point kinetics equations with one group of delayed neutrons. An adequately chosen constant exchange coefficient has been added to take account of the transfer of neutrons between meshes. In addition, the iodine and xenon concentrations are calculated by a multipoint iodine-xenon estimator. The rise in coolant temperature, from the bottom to the top of each mesh, is given by a first order differential equation and is proportional to the amount of power generated by nuclear fission. In each mesh, the growth rate of the fission chain reaction is characterized by a quantity known as reactivity [17, 18]. Criticality is achieved when the fission chain reaction is stable and self-sustaining, i.e., when the reactivity of the reactor core equals zero. Reactivity depends on many factors, namely the fuel and coolant temperatures, the xenon concentration of the reactor core, the position of the control rods and the boron concentration of the primary coolant. The resulting feedbacks and underlying interactions between all involved variables lead to strong plant nonlinearities and coupling.

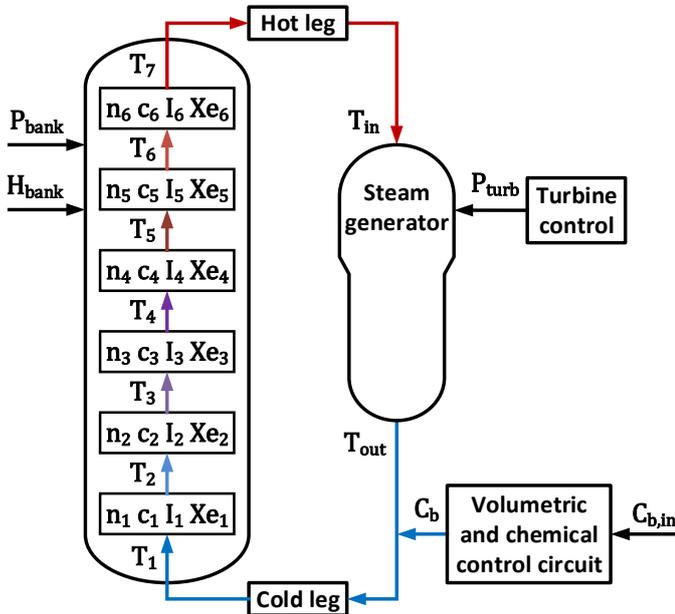


Fig. 1: Simplified diagram of the multipoint PWR core kinetic model

In order to move from reactor core to plant scale, a very simple steam generator is modeled. Once again, it is assumed that the fall in temperature between the inlet and outlet of the steam generator is given by a first order

differential equation and is proportional to the power of the turbine. In fact, whenever the power of the turbine is decreased (resp. increased), the average coolant temperature will increase (resp. decrease). As a result, the nuclear power of the reactor core will also decrease (resp. increase) and ultimately reach steady-state because the PWR was designed so that the fission chain reaction can be stabilized by temperature reactivity feedbacks alone (i.e., Doppler effect and moderation) [17]. However, it is very likely that, without control, the average coolant temperature will deviate from the limiting conditions of operation. Furthermore, xenon-induced spatial power oscillations may appear if the axial power distribution of the reactor core is left uncontrolled. Therefore, the role of the core control system is to ensure that both the ACT and the AO stay within their respective limits before, during and after power variations [4]. This can be achieved by slightly altering the reactivity of the reactor core.

In mode T [5], the nuclear reaction is controlled by moving two separate banks of control rods inside the reactor core and by adjusting the boron concentration of the primary coolant. Reactivity can be increased (resp. decreased) either by withdrawing (resp. inserting) the control rods from (resp. in) the reactor core or by decreasing (resp. increasing) the boron concentration of the coolant. The control rods are mainly used to handle abrupt changes in reactivity but adversely impact the power distribution of the reactor core. In fact, reactivity will mostly decrease in the region of the reactor core where the control rods are being inserted. To alleviate this problem, one bank of control rods, denoted by  $P_{\text{bank}}$ , is dedicated to ACT regulation whereas the other, denoted by  $H_{\text{bank}}$ , is dedicated to AO regulation. Conversely, the boron concentration of the primary coolant acts uniformly on the reactor core but takes time to become effective. This is due to the fact that the solutions of boric acid and demineralized water, which are injected to modify the boron concentration, have to flow throughout the volumetric and chemical control circuit before reaching the coolant. Hence, boron concentration adjustments are mostly used to counterbalance axial xenon oscillations after power variations. To summarize:

- ACT regulation is achieved by moving  $P_{\text{bank}}$
- AO regulation is achieved by moving  $H_{\text{bank}}$  during power variations and by adjusting the boron concentration of the primary coolant during xenon oscillations

In addition, the mode T allows the reactor core to quickly return to its nominal power. This is achieved by controlling the level of insertion of  $P_{\text{bank}}$  in the reactor core. Last but not least, all three controlled variables can be successfully regulated even when the turbine provides frequency control to the grid.

## 2.2. Control Objectives

As mentioned before, the goal of an advanced core control system is to keep the ACT, the AO and the position

of  $P_{bank}$  within appropriate limits during power variations and while frequency control is active. These limits were determined beforehand by nuclear reactor engineers to comply with genuine performance and safety requirements. In mode T, their values are given by:

$$\begin{aligned} |T_a(t) - T_{a,ref}(P_{ref}(t))| &\leq 1.5 \text{ }^\circ\text{C} \\ |AO(t) - AO_{ref}| &\leq 5 \% \\ |P_{bank}(t) - P_{bank,ref}(P_{ref}(t))| &\leq 30 \text{ steps,} \end{aligned} \quad (1)$$

where  $T_a$  is the average temperature of the coolant,  $AO$  is the axial offset of the reactor core,  $P_{bank}$  is the position of the first bank of control rods, and  $T_{a,ref}$ ,  $AO_{ref}$  and  $P_{bank,ref}$  are their respective reference signals. Note that the reference signals  $T_{a,ref}$  and  $P_{bank,ref}$  are both piecewise linear functions of the load profile  $P_{ref}$  whereas  $AO_{ref}$  is a constant. Shorthand notation for the deviations of ACT, AO and  $P_{bank}$  are:

$$\begin{aligned} \Delta T_a(t) &= T_a(t) - T_{a,ref} \\ \Delta AO(t) &= AO(t) - AO_{ref} \\ \Delta P_{bank}(t) &= P_{bank}(t) - P_{bank,ref}(P_{ref}(t)). \end{aligned}$$

Actuator saturation should also be considered to make the core control system practically workable. In our case, the reactor core is actuated by changing the speeds of both  $P_{bank}$  and  $H_{bank}$ , and by adjusting the boron concentration of the coolant. The speeds of  $P_{bank}$  and  $H_{bank}$  are simply limited by:

$$\begin{aligned} \left| \frac{dP_{bank}(t)}{dt} \right| &\leq 75 \text{ steps/min} \\ \left| \frac{dH_{bank}(t)}{dt} \right| &\leq 75 \text{ steps/min,} \end{aligned} \quad (2)$$

where  $H_{bank}$  is the position of the second bank of control rods. Besides,  $P_{bank}$  and  $H_{bank}$  are saturated as well since the control rods cannot move beyond their maximum insertion and extraction thresholds:

$$\begin{aligned} 36 &\leq P_{bank} \leq 1053 \text{ steps} \\ 9 &\leq H_{bank} \leq 411 \text{ steps.} \end{aligned} \quad (3)$$

The upper and lower bounds of the injected boron concentration variation rate depend on the current boron concentration of the coolant. More precisely, the higher the boron concentration of the coolant, the lower the efficiency of a boric acid injection. Similarly, the lower the boron concentration of the coolant, the lower the efficiency of a demineralized water injection. This translates into the following inequalities:

$$-\frac{10}{M_t} C_b(t) \leq \frac{dC_{b,in}(t)}{dt} \leq \frac{3}{M_t} (C_{b,max} - C_b(t)), \quad (4)$$

where  $C_{b,max}$  is the maximum boron concentration of the coolant,  $M_t$  is the total mass of water in the primary cooling circuit,  $C_{b,in}$  is the injected boron concentration and  $C_b$  is the boron concentration of the coolant.

In our model, it is assumed that the turbine control system has already been designed and is functioning perfectly. Thus, the power of the turbine can be expressed as  $P_{turb} = P_{ref} + \delta P_{ref}$  where  $P_{ref}$  is the known load profile and  $\delta P_{ref}$  is an unknown reference signal that is added when frequency control is active. Specifically,  $P_{ref}$  appears as a ramp-like signal during power variations and as a step-like signal otherwise. The additional term  $\delta P_{ref}$  is norm-bounded by 2.5 %NP and changes about every second. From the perspective of the reactor core,  $P_{turb}$  is seen as a disturbance signal that has to be rejected by the core control system.

### 3. Fixed-Structure Gain-Scheduled Controller

The idea of gain scheduling is to break down the nonlinear control problem into a finite number of linear sub-problems [19, 20]. The main advantage of this divide-and-conquer approach is to rely on well-known linear design tools rather than convoluted nonlinear methods. This is particularly interesting given the recent developments in non-smooth optimization for structured robust control [21].

#### 3.1. Linearization of the Nonlinear Model

The first step in designing the fixed-structure  $H_2/H_\infty$  gain-scheduled controller is to linearize the nonlinear model of the reactor core around several operating points. All were determined using Framatome's certified three-dimensional core kinetics computer code, named SMART [22], starting from stationary conditions with equilibrium xenon at 100 %NP. The computation of an operating point begins by setting the nuclear power of the reactor core at a constant level. Then, the positions of  $P_{bank}$  and  $H_{bank}$  are updated so that criticality is achieved with minimal ACT and AO deviations. The boron concentration of the coolant, on the other hand, is fixed according to the fuel burn-up. In total, 21 operating points were computed over an evenly spaced grid of power values between 100 %NP and 50 %NP (i.e., one point every 2.5 %NP). This number of points is sufficient to obtain a fine-grained description of the original system and to ensure a smooth transition of the gains.

Then, the operating points are uniquely defined by a set of scheduling variables that covers the whole operating range of the real plant. Initially, two different scheduling variables were considered: the position  $P_{bank}$  and the known load profile  $P_{ref}$ . However, using both variables is irrelevant here, as the closed-loop dynamic of  $P_{bank}$  should be directly linked to  $P_{ref}$ . The question that remains is whether to schedule the controller with respect to the internal variable  $P_{bank}$  or the exogenous signal  $P_{ref}$ . Theoretically speaking, it would be preferable to choose  $P_{ref}$  over  $P_{bank}$  since the impact of the hidden coupling terms, which appear in the dynamic of the linearized gain-scheduled controller [20], is less pronounced when an external signal is used for scheduling. Yet, in reality, the

337 behavior of the reactor core depends heavily on the position  
 338 of the control rods. Therefore, just like in [8],  $P_{bank}$   
 339 is selected as the only scheduling variable.

340 Subsequently, a collection of 21 linear open-loop models  
 341 is numerically computed with the linearization algorithm  
 342 of MATLAB<sup>®</sup>/Simulink<sup>®</sup>. For all  $i \in \llbracket 1, 21 \rrbracket$ , the  $i$ -th  
 343 linear open-loop model is obtained by linearizing the non-  
 344 linear model of the plant around the operating point de-  
 345 fined by  $\sigma_i = P_{bank,eq}^{(i)}$ . The iodine-xenon estimator has  
 346 been discarded from the linearization process because its  
 347 dynamics are much slower than that of the controlled out-  
 348 puts. **Finally**, the state-space realization of the  $i$ -th linear-  
 349 ized open-loop model is given by:

$$\begin{aligned} \dot{x}_r(t) &= A_i x_r(t) + B u(t) + E d(t) \\ y &= C x_r(t), \end{aligned}$$

351 where  $x_r \in \mathbb{R}^{25}$  is the truncated state vector (i.e., without  
 352 iodine and xenon dynamics),  $u = [\dot{P}_{bank}, \dot{H}_{bank}, \dot{C}_{b,in}]^T \in$   
 353  $\mathbb{R}^3$  are the control inputs,  $d = P_{turb} \in \mathbb{R}$  is the exogenous  
 354 disturbance, and  $y = [T_{in}, T_{out}, AO, P_{bank}, P_{core}]^T \in \mathbb{R}^5$   
 355 are the measured outputs. Note that the controlled out-  
 356 puts  $z = [T_a, AO, P_{bank}]^T \in \mathbb{R}^3$  are a subset of the mea-  
 357 sured output  $y$  in the sense that:

$$z = T y \quad \text{with} \quad T = \begin{bmatrix} 1/2 & 1/2 & 0 & 0 & 0 \\ 0 & 0 & 1 & 0 & 0 \\ 0 & 0 & 0 & 1 & 0 \end{bmatrix}.$$

359 Since the control inputs  $u$  are the time derivatives of the  
 360 state variables  $P_{bank}$ ,  $H_{bank}$  and  $C_{b,in}$ , the open-loop state  
 361 matrix  $A_i$  and the control matrix  $B$  can be decomposed  
 362 as:

$$A_i = \begin{bmatrix} A_{1i} & A_{2i} \\ \mathbf{0} & \mathbf{0} \end{bmatrix}, \quad B = \begin{bmatrix} \mathbf{0} \\ I_3 \end{bmatrix}.$$

364 Thus, the poles of the  $i$ -th linearized open-loop model are  
 365 the union of the eigenvalues of  $A_{1i}$  plus three controllable  
 366 poles located at  $s = 0$ . **For the chosen set of model param-**  
 367 **eters, it was observed that every matrix  $A_{1i}$  is Hurwitz**  
 368 **stable. This numerical assessment is consistent with reality**  
 369 **insofar as the reactor core was designed to be stable**  
 370 **under normal conditions.** Hence, it can be concluded that  
 371 the linear open-loop models are all stabilizable.

### 372 3.2. Architecture of the Controller

373 To achieve asymptotic tracking of a class of reference  
 374 inputs in the presence of another class of disturbances, a  
 375 model of both exogenous signals should be embedded in  
 376 the controller. Then, this model should be duplicated as  
 377 many times as the number of outputs to be controlled [23].  
 378 Here, the reference inputs and the disturbance belong to  
 379 the same class of signals that are constant and piecewise  
 380 linear. Thus, the controller should comprise at least two  
 381 integrators per controlled output channel to achieve the  
 382 desired closed-loop performance. However, an integrator  
 383 is already included in every input-output transfer func-  
 384 tion of the linearized open-loop models. Hence, only one

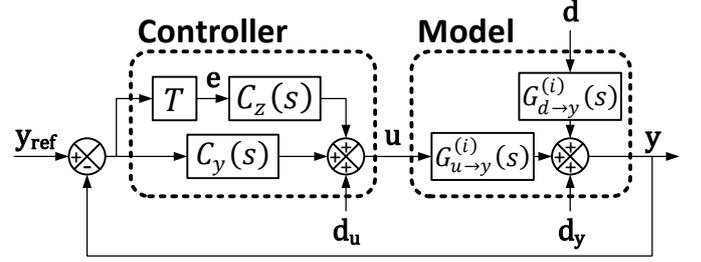


Fig. 2: Block diagram of the linear time-invariant closed-loop system computed at the  $i$ -th operating point

385 extra integrator needs to be added per controlled output  
 386 channel. This can be realized with the following linear  
 387 time-invariant output feedback controller:

$$\begin{aligned} \dot{x}_I(t) &= K_I (z_{ref}(t) - z(t)) \\ u(t) &= K_P (y_{ref}(t) - y(t)) + x_I(t), \end{aligned}$$

389 where  $y_{ref} \in \mathbb{R}^5$  are the reference signals of the measured  
 390 outputs and  $z_{ref} \in \mathbb{R}^3$  are the reference signals of the con-  
 391 trolled outputs. This controller can be seen as a multiple-  
 392 input multiple-output version of a Proportional-Integral  
 393 controller with gain matrices  $K_P \in \mathbb{R}^{3 \times 5}$  and  $K_I \in \mathbb{R}^{3 \times 3}$ .  
 394 The proportional part of the controller is then filtered to  
 395 provide robustness against high-frequency noise and un-  
 396 modeled dynamics:

$$\begin{aligned} \dot{x}_I(t) &= K_I (z_{ref}(t) - z(t)) \\ \dot{x}_P(t) &= \text{diag}\left(\frac{1}{a}, \frac{1}{a}, \frac{1}{b}\right) \left(-x_P(t) + K_P (y_{ref}(t) - y(t))\right) \\ u(t) &= x_P(t) + x_I(t), \end{aligned}$$

398 where  $a > 0$  and  $b > 0$  are the time constants of the first-  
 399 order low-pass filters that are applied to  $x_P$ . The time  
 400 constants of the first and second filters are chosen equal  
 401 since the dynamic of  $P_{bank}$  is expected to be similar to that  
 402 of  $H_{bank}$ . A block diagram of the linear time-invariant  
 403 closed-loop system that is computed at the  $i$ -th operating  
 404 point,  $i \in \llbracket 1, 21 \rrbracket$ , is shown in Fig. 2, where:

$$\begin{aligned} G_{d \rightarrow y}^{(i)}(s) &= C (sI_{25} - A_i)^{-1} E \\ G_{u \rightarrow y}^{(i)}(s) &= C (sI_{25} - A_i)^{-1} B \\ C_y(s) &= \text{diag}\left(\frac{1}{as+1}, \frac{1}{as+1}, \frac{1}{bs+1}\right) K_P \\ C_z(s) &= \frac{1}{s} K_I, \end{aligned}$$

and  $e = z - z_{ref}$  is the tracking error:

$$e = [\Delta T_a, \Delta AO, \Delta P_{bank}]^T. \quad (5)$$

408 The signals  $d_u \in \mathbb{R}^3$  and  $d_y \in \mathbb{R}^5$  are unknown additive  
 409 disturbances that may appear on the actuators and on the  
 410 sensors of the plant. The input and the output sensitivity  
 411 functions:

$$\begin{aligned} S_u^{(i)}(s) &= \left[ I_5 + (C_y(s) + T C_z(s)) G_{u \rightarrow y}^{(i)}(s) \right]^{-1} \\ S_y^{(i)}(s) &= \left[ I_3 + G_{u \rightarrow y}^{(i)}(s) (C_y(s) + T C_z(s)) \right]^{-1}, \end{aligned}$$

413 are given by the closed-loop transfer functions from  $d_u$  to  
414  $u$  and from  $d_y$  to  $y$  respectively.

### 415 3.3. Tuning and Gain-Scheduling

416 The conventional approach [19, 20] for scheduling the  
417 gains of the controller would be to: 1) synthesize a linear  
418 time-invariant controller at each operating point using,  
419 e.g., a fixed-structure  $H_2/H_\infty$  synthesis algorithm and 2)  
420 smoothly interpolate the gains of the controller as a function  
421 of the scheduling variable  $P_{bank}$ . However, independently  
422 tuning the controller at each operating point may  
423 lead to undesirable jumps in the values of the gains once  
424 they have been interpolated. To prevent this, the gains can  
425 be tuned against multiple neighboring models rather than  
426 just one [8]. Here, the tuning and scheduling stages are  
427 blended together using the Gain Surface Tuning method  
428 presented in [13]. First, the gain matrices  $K_P$ ,  $K_I$  and the  
429 time constants  $a$ ,  $b$  are written as quadratic polynomials  
430 of the scheduling variable  $\sigma = P_{bank}$ :

$$\begin{aligned} K_P(\sigma) &= K_{P0} + K_{P1}\sigma + K_{P2}\sigma^2 \\ K_I(\sigma) &= K_{I0} + K_{I1}\sigma + K_{I2}\sigma^2 \\ a(\sigma) &= a_0 + a_1\sigma + a_2\sigma^2 \\ b(\sigma) &= b_0 + b_1\sigma + b_2\sigma^2, \end{aligned}$$

432 with coefficients  $K_{Pj} \in \mathbb{R}^{3 \times 5}$ ,  $K_{Ij} \in \mathbb{R}^{3 \times 3}$ ,  $a_j \in \mathbb{R}$  and  
433  $b_j \in \mathbb{R}$  for all  $j \in \{1, 2, 3\}$ . Then, these coefficients are  
434 tuned to ensure proper closed-loop performance at each  
435 operating point. Specifically, let  $\mathbf{K} \in \mathbb{R}^{78}$  be the vector of  
436 tunable parameters. For each linearized open-loop model,  
437 the performance objectives are represented by the following  
438 frequency-domain criteria:

$$\begin{aligned} \min_{\mathbf{K}} \quad & \left\| \text{diag}(\beta_1, \beta_2, \beta_3) H_{z_{ref} \rightarrow e}^{(i)}(\mathbf{K}, s) \frac{1}{s^2} \right\|_2 \\ \text{subject to:} \quad & \begin{cases} \left\| W_{11} H_{d \rightarrow e_1}^{(i)}(\mathbf{K}, s) \right\|_\infty \leq 1 \\ \left\| W_{12} H_{d \rightarrow e_2}^{(i)}(\mathbf{K}, s) \right\|_\infty \leq 1 \\ \left\| W_{13} H_{d \rightarrow e_3}^{(i)}(\mathbf{K}, s) \right\|_\infty \leq 1 \\ \left\| W_{21} H_{d \rightarrow u_1}^{(i)}(\mathbf{K}, s) \right\|_\infty \leq 1 \\ \left\| W_{22} H_{d \rightarrow u_2}^{(i)}(\mathbf{K}, s) \right\|_\infty \leq 1 \\ \left\| W_{23} H_{d \rightarrow u_3}^{(i)}(\mathbf{K}, s) \right\|_\infty \leq 1 \\ \left\| \gamma H_{d_u \rightarrow u}^{(i)}(\mathbf{K}, s) \right\|_\infty \leq 1 \\ \text{Re}(p^{(i)}(\mathbf{K})) \leq -\alpha, \end{cases} \end{aligned}$$

440 where, for all  $i \in \llbracket 1, 21 \rrbracket$ ,  $H_{u \rightarrow y}^{(i)}$  denotes the closed-loop  
441 transfer function from input signal  $u$  to output signal  $y$  for  
442 the  $i$ -th plant model,  $\|\cdot\|_2$  and  $\|\cdot\|_\infty$  are the  $H_2$  and  $H_\infty$   
443 norms, and  $p^{(i)}$  are the closed-loop poles of the  $i$ -th plant  
444 model. Having two integrators in the loop transfer function  
445 means that  $H_{z_{ref} \rightarrow e}^{(i)}$  has two zeros at  $s = 0$ . Hence, its

$H_2$  norm can be shaped with a double integrator weighting  
446 function which, in this context, is interpreted as a refer-  
447 ence signal generator. In fact, minimizing the frequency-  
448 weighted  $H_2$  norm of  $H_{z_{ref} \rightarrow e}^{(i)}$  ensures both asymptotic  
449 tracking and disturbance rejection of step-like and ramp-  
450 like exogenous signals. The parameters  $(\beta_1, \beta_2, \beta_3) \in \mathbb{R}_{>0}^3$   
451 are chosen such that ACT regulation takes precedence over  
452 AO regulation which, in turn, takes precedence over  $P_{bank}$   
453 regulation. The scalar weights  $(W_{11}, W_{12}, W_{13}) \in \mathbb{R}_{>0}^3$  and  
454  $(W_{21}, W_{22}, W_{23}) \in \mathbb{R}_{>0}^3$  are used to limit the maximum set-  
455 point deviation and the maximum control effort that are  
456 induced by a change in the power of the turbine. The reason  
457 why the controller is tuned against several single-input,  
458 single-output  $H_\infty$  constraints, rather than one multiple-  
459 input, multiple-output  $H_\infty$  constraint, is to make the opti-  
460 mization problem more readable. Finally, the parameters  
461  $\gamma \in \mathbb{R}_{>0}$  and  $\alpha \in \mathbb{R}_{>0}$  are selected to provide a sufficient  
462 level of input-robustness and stability to the closed-loop  
463 system. 464

## 465 4. Nonlinear Model Predictive Controller

The principle of model predictive control is to use a  
466 model of the plant to predict and optimize its future be-  
467 havior by repeatedly solving an on-line optimal control  
468 problem over a finite horizon [14, 15]. This control tech-  
469 nique, also known as receding horizon control, has proven  
470 to be very successful in the process industry because it  
471 can handle large scale, multi-input multi-output nonlin-  
472 ear systems that are subject to actuator and state con-  
473 straints [24]. 474

### 475 4.1. Model Reduction

The nonlinear state-space representation of the full-  
476 order PWR model can be written as:

$$\begin{aligned} \dot{x}(t) &= f_0(x(t), u(t), d(t)) \\ x(t_0) &= x_0, \end{aligned}$$

477 where  $x \in \mathbb{R}^{37}$  is the full state vector,  $x_0 \in \mathbb{R}^{37}$  is the initial  
478 state,  $u \in \mathbb{R}^3$  are the control inputs, and  $d \in \mathbb{R}$  is the ex-  
479 ogenous disturbance. In the sequel, it is assumed that the  
480 state is perfectly measured. Since the model is intended  
481 to be used in a model predictive control framework, special  
482 attention is given to its computational complexity. In  
483 practice, many numerical simulation runs are performed  
484 to solve the optimal control problem at each time step.  
485 However, the multiple time scale behavior of the model,  
486 ranging from fast neutron to slow xenon dynamics, ren-  
487 ders it numerically stiff [25]. Although the full-order model  
488 can be simulated using powerful integration solvers, such  
489 as [26] or [27], model reduction is still conducted to make  
490 the optimal control problem more tractable. State estima-  
491 tion, which is outside the scope of the paper, is another  
492 motivation for reducing the order of the model. In the  
493 future, a state observer should be designed to implement  
494 the NMPC controller [28]. This task would obviously be  
495 496

547 easier to achieve with a reduced-order model. Therefore, 537  
548 singular perturbation theory [29] is employed to remove 538  
549 the dynamics that are not essential regarding the control 539  
550 objectives. A well-known approximation in nuclear reactor 540  
551 physics is the prompt-jump approximation [17, 18]. 541  
552 Whenever a fission reaction occurs, two types of neutrons 542  
553 are emitted: prompt neutrons, which are directly emitted 543  
554 from fission, and delayed neutrons, which are emitted 544  
555 a moment later from the radioactive decay of fission 545  
556 products. Under normal operating conditions, the reactor 546  
557 core is predominantly governed by the dynamic of delayed 547  
558 neutrons. Consequently, prompt neutron dynamics 548  
559 can be reasonably approximated by steady-state equations 549  
560 without any significant loss of accuracy [30]. The result- 550  
561 ing reduced-order model is given by a set of semi-explicit 551  
562 differential-algebraic equations:

$$513 \begin{aligned} \dot{x}_d(t) &= f(x_d(t), x_a(t), u(t), d(t)) \\ 0 &= g(x_d(t), x_a(t), u(t), d(t)) \\ x_d(0) &= x_{d0}, \end{aligned}$$

514 where the state is divided into a slow differential part  $x_d \in$   
515  $\mathbb{R}^{31}$  and a fast algebraic part  $x_a \in \mathbb{R}^6$ .

#### 516 4.2. Formulation of the Optimal Control Problem

517 At each time step, the NMPC algorithm yields a se- 517  
518 quence of control inputs by solving a finite horizon optimal 518  
519 control problem that is initialized with the current state of 519  
520 the plant. Usually, only the first component of the control 520  
521 input sequence is applied open-loop to the plant until a 521  
522 new one is calculated at the next time step. The greatest 522  
523 strength of NMPC is that all the previously defined core 523  
524 control specifications can be directly incorporated into the 524  
525 finite horizon optimal control problem:

$$\min_{\bar{u}(\cdot)} \int_t^{t+NT_s} \left( \|e(\bar{x}_d(\tau), \bar{x}_a(\tau), \bar{d}(\tau))\|_Q^2 + \|\bar{u}(\tau)\|_R^2 \right) d\tau$$

subject to:

$$\forall \tau \in [t, t + NT_s] \begin{cases} c(\bar{x}_d(\tau), \bar{x}_a(\tau), \bar{u}(\tau), \bar{d}(\tau)) \leq 0 \\ \dot{\bar{x}}_d(\tau) = f(\bar{x}_d(\tau), \bar{x}_a(\tau), \bar{u}(\tau), \bar{d}(\tau)) \\ 0 = g(\bar{x}_d(\tau), \bar{x}_a(\tau), \bar{u}(\tau), \bar{d}(\tau)) \\ \bar{x}_d(t) = x_d(t), \end{cases} \quad (6)$$

526 where  $N \geq 1$  is the length of the prediction horizon,  $T_s > 0$   
527 is the sampling time of the controller, and  $x_d(t)$  is the dif- 527  
528 ferential state of the plant that is measured at time  $t \geq 0$ . 528  
529 The core control specifications (1) – (4) are represented by 529  
530 the inequality constraints map  $c: \mathbb{R}^{31} \times \mathbb{R}^6 \times \mathbb{R}^3 \times \mathbb{R} \rightarrow \mathbb{R}^{16}$   
531 and the tracking error (5) is given by the nonlinear map 531  
532  $e: \mathbb{R}^{31} \times \mathbb{R}^6 \times \mathbb{R} \rightarrow \mathbb{R}^3$ . The weighted 2-norms  $\|e\|_Q$  and 532  
533  $\|u\|_R$  are defined by  $\|e\|_Q^2 := e^T Q e$  and  $\|u\|_R^2 := u^T R u$   
534 where  $Q \in \mathbb{R}^{3 \times 3}$  and  $R \in \mathbb{R}^{3 \times 3}$  are symmetric positive- 534  
535 definite matrices. The bar subscript denotes internal 535  
536

model variables that are used within the controller to pre- 537  
538 dict the behavior of the real plant. In particular, the dis- 538  
539 turbance signal is written as  $d(t) = \bar{d}(t) + \delta d(t)$  where  $\bar{d}$  539  
540 is the known load profile that is sent in advance by the 540  
541 grid dispatcher to the plant and  $|\delta d| \leq 2.5 \%$ NP is the un- 541  
542 known norm-bounded term that appears when frequency 542  
543 control is active. 543

Transcription of the continuous optimal control problem 544  
545 into a finite-dimensional nonlinear programming problem 545  
546 is commonly achieved using either direct single-shooting or 546  
547 direct multiple-shooting [31, 32]. Here, both options will 547  
548 be implemented and tested thereafter. In a direct method, 548  
549 the continuous-time control inputs are approximated with 549  
550 a finite number of parameters. Most of the time, the control 550  
551 input sequence  $\bar{\mathbf{u}} = \{\bar{u}[0], \dots, \bar{u}[N-1]\}$  is discretized 551  
552 with a piecewise constant parameterization: 552

$$\forall k \in \llbracket 0, N-1 \rrbracket, \forall \tau \in [t_k, t_{k+1}], \bar{u}(\tau) = \bar{u}[k],$$

where  $t_k = t + kT_s$ . The key difference between single- 554  
555 shooting and multiple-shooting is the parameterization of 555  
556 the state trajectory. 556

In single-shooting, the state trajectory is computed in 557  
558 one shot, from  $t_0$  to  $t_N$ , outside of the optimization prob- 558  
559 lem. In fact, for a given control input sequence  $\bar{\mathbf{u}}$  and a 559  
560 fixed initial state  $x_d(t)$ , the state trajectory can be entirely 560  
561 determined by simulating the model of the plant: 561

$$\forall k \in \llbracket 0, N-1 \rrbracket, \begin{pmatrix} \bar{x}_d(t_{k+1}) \\ \bar{x}_a(t_{k+1}) \end{pmatrix} = \phi_{ss}(t_{k+1}; x_d(t), \bar{\mathbf{u}}),$$

where  $\phi_{ss}: \mathbb{R} \times \mathbb{R}^{31} \times \mathbb{R}^{3 \times N} \rightarrow \mathbb{R}^{37}$  is a numerical integra- 563  
564 tion operator that returns the solution of the initial value 564  
565 problem: 565

$$\forall \tau \in [t_0, t_N] \begin{cases} \dot{\bar{x}}_d(\tau) = f(\bar{x}_d(\tau), \bar{x}_a(\tau), \bar{u}(\tau), \bar{d}(\tau)) \\ 0 = g(\bar{x}_d(\tau), \bar{x}_a(\tau), \bar{u}(\tau), \bar{d}(\tau)) \\ \bar{x}_d(t_0) = x_d(t), \end{cases}$$

at time  $t_{k+1}$ . Therefore, the optimization problem can be 567  
568 reduced to: 568

$$\min_{\bar{\mathbf{u}}} \sum_{k=0}^{N-1} \|e(\phi_{ss}(t_{k+1}; x_d(t), \bar{\mathbf{u}}), \bar{d}(t_{k+1}))\|_Q^2 + \|\bar{u}[k]\|_R^2$$

subject to:

$$\forall k \in \llbracket 0, N-1 \rrbracket, c(\phi_{ss}(t_{k+1}; x_d(t), \bar{\mathbf{u}}), \bar{u}[k], \bar{d}(t_{k+1})) \leq 0.$$

Hence, the advantage of using single-shooting is to remove 570  
571 the equality constraints from the optimization problem. 571  
572 Moreover, the state trajectory is always guaranteed to 572  
573 be consistent with the dynamical equations of the model. 573  
574 However, using single-shooting can increase the computa- 574  
575 tional complexity of the optimization problem because the 575  
576 state equations are repeatedly propagated into both the 576  
577 cost function and the constraints. This is especially true 577  
578 for nonlinear and/or unstable systems. 578

In multiple-shooting, the state trajectory is cut into 579  
580  $N$  pieces which are independently computed over each 580

time interval  $[t_k, t_{k+1}]$ . These pieces are then **reassembled** within the optimization problem by imposing equality constraints on the differential and algebraic states:

$$\min_{\bar{\mathbf{x}}_d, \bar{\mathbf{x}}_a, \bar{\mathbf{u}}} \sum_{k=0}^{N-1} \|e(\bar{x}_d[k+1], \bar{x}_a[k], \bar{d}(t_{k+1}))\|_Q^2 + \|\bar{u}[k]\|_R^2$$

subject to:

$$\forall k \in \llbracket 0, N-1 \rrbracket,$$

$$\begin{cases} c(\bar{x}_d[k+1], \bar{x}_a[k], \bar{u}[k], \bar{d}(t_{k+1})) \leq 0 \\ \begin{pmatrix} \bar{x}_d[k+1] \\ \bar{x}_a[k] \end{pmatrix} - \phi_{ms}(t_{k+1}; \bar{x}_d[k], \bar{u}[k]) = 0 \\ \bar{x}_d[0] - x_d(t) = 0, \end{cases}$$

where  $\phi_{ms}: \mathbb{R} \times \mathbb{R}^{31} \times \mathbb{R}^6 \rightarrow \mathbb{R}^{37}$  is a numerical integration operator that returns the solution of the initial value problem:

$$\forall \tau \in [t_k, t_{k+1}] \begin{cases} \dot{\bar{x}}_d(\tau) = f(\bar{x}_d(\tau), \bar{x}_a(\tau), \bar{u}(\tau), \bar{d}(\tau)) \\ 0 = g(\bar{x}_d(\tau), \bar{x}_a(\tau), \bar{u}(\tau), \bar{d}(\tau)) \\ \bar{x}_d(t_k) = x_d(t_k), \end{cases}$$

at time  $t_{k+1}$ . The drawback of multiple-shooting is that the dimension of the optimization problem is **substantially** increased since the differential and algebraic state sequences  $\bar{\mathbf{x}}_d = \{\bar{x}_d[0], \dots, \bar{x}_d[N]\}$  and  $\bar{\mathbf{x}}_a = \{\bar{x}_a[0], \dots, \bar{x}_a[N-1]\}$  become decision variables. However, this is not a real issue because the structure of the resulting nonlinear program is block-sparse, provided that the decision variables are arranged as follows:

$$(\bar{x}_d[0]^T, \bar{u}[0]^T, \bar{x}_a[0]^T, \dots, \bar{x}_d[N-1]^T, \bar{u}[N-1]^T, \bar{x}_a[N-1]^T, \bar{x}_d[N]^T)^T.$$

In fact, it has often been reported that efficient sparsity exploitation can actually improve the rate of convergence of the Newton-type algorithm which is used to solve the nonlinear program [33]. **Furthermore**, having  $\bar{\mathbf{x}}_d$  and  $\bar{\mathbf{x}}_a$  as decision variables is beneficial for initializing the problem since *a priori* information about the state trajectory can be supplied to the algorithm. Last but not least, multiple-shooting is better suited for highly nonlinear and/or unstable plants because integration is broken down into short time intervals, meaning that the nonlinearity is equally distributed over the shooting nodes.

#### 4.3. Practical Implementation Issues

The nonlinear program that is transcribed using single shooting is composed of  $3N$  decision variables and  $16N$  inequality constraints. On the other hand, the nonlinear program that is transcribed using multiple-shooting is composed of  $40N+31$  decision variables,  $37N+31$  equality constraints and  $16N$  inequality constraints. In both cases, it is expected that the resolution time of the optimization problem becomes greater than the duration  $T_s$  in between

two sampling instants because the length  $N \geq 1$  of the prediction horizon has to be large enough to ensure closed-loop stability. Yet, we still want to recalculate the control inputs at each time step so the plant does not remain in open-loop for too long. Therefore, the optimization routine is deliberately terminated once the resolution time exceeds  $T_s$  seconds. Even if the solution is sub-optimal [34], we believe that it should have sufficiently converged to provide an acceptable level of closed-loop performance.

**Consequently**, a prediction-based delay compensation scheme [35] is incorporated into the NMPC algorithm to account for the non-negligible resolution time  $0 < \tau \leq T_s$  of the optimization problem. To illustrate, let  $\mu_N(x_d(t))$  be the feedback value given by the (sub-optimal) control input sequence  $\bar{\mathbf{u}}^*(x_d(t))$  that (almost) solves the optimal control problem with initial state  $x_d(t)$ . Because of the computational delay,  $\mu_N(x_d(t))$  is applied to the plant at time  $t+\tau$  instead of time  $t$ . Thus, the greater the mismatch between  $x_d(t)$  and  $x_d(t+\tau)$ , the greater the difference between the expected and the real behavior of the closed-loop system. Knowing the maximum resolution time  $\tau_{max} = T_s$  of the optimization problem, the idea of [35] is to solve the optimal control problem in advance, starting from an estimate  $\bar{x}_d(t+T_s)$  of the future state in place of the currently measured state  $x_d(t)$ . The resulting feedback value  $\mu_N(\bar{x}_d(t+T_s))$ , available at time  $t+\tau$ , is then applied to the plant accordingly at time  $t+T_s$ . The next state estimate can be computed straightforwardly by simulating the model of the plant from the newly measured state  $x_d(t+T_s)$  with input  $\mu_N(\bar{x}_d(t+T_s))$ . Note that, since the state is estimated in an open-loop fashion, it is almost certain that the real state  $x_d(t+T_s)$  will differ from  $\bar{x}_d(t+T_s)$ . However, it is still relevant to use the delay-compensated input, as some degree of robustness is expected against estimation errors [36].

## 5. Simulation Results

### 5.1. Validation Scenario

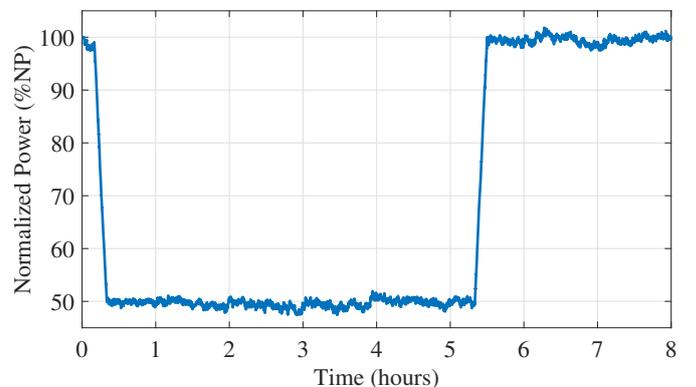


Fig. 3: Normalized power of the turbine with frequency control

To assess and compare the performance of the two core control systems, a typical 8-hour load-following

660 scenario is conducted on PWRSimu, an intermediate  
 661 complexity PWR simulator developed by Framatome in  
 662 MATLAB<sup>®</sup>/Simulink<sup>®</sup>. As shown in Fig. 3, this sce-  
 663 nario involves two power variations (from 100 %NP to  
 664 50 %NP and *vice versa*) that are carried out at a rate of  
 665 5 %NP/min with frequency control. The full-order non-  
 666 linear PWR model is simulated on Simulink with the stiff  
 667 adaptive-step integration solver `ode15s` [27].

668 The fixed-structure gain-scheduled controller has been  
 669 tuned with the following set of parameters:

$$\begin{aligned} \beta_1 &= 10/1.5 & \beta_2 &= 5/5 & \beta_3 &= 2/30 \\ W_{11} &= 10/1.5 & W_{12} &= 10/5 & W_{13} &= 10/30 \\ W_{21} &= 10/1.25 & W_{22} &= 10/1.25 & W_{23} &= 10/0.04 \\ \gamma &= 0.5 & \alpha &= 0.0001, \end{aligned}$$

671 using `Systune` (MATLAB Control System Toolbox<sup>™</sup>) [21].  
 672 In order to model the whole closed-loop system in  
 673 Simulink, the quadratic polynomials  $K_P(\sigma)$ ,  $K_I(\sigma)$   
 674 and  $b(\sigma)$  are approximated by feeding the scheduling vari-  
 675 able  $\sigma = P_{bank}$  into 1-D lookup table blocks with linear  
 676 interpolation and clip extrapolation options.

677 Tuning of the NMPC controller has been achieved in two  
 678 steps. First, the matrices  $Q$  and  $R$  of the cost function (6)  
 679 are selected diagonal with normalized weights:

$$\begin{aligned} q_1 &= \frac{\varepsilon_1}{(e_{1,max})^2} & q_2 &= \frac{\varepsilon_2}{(e_{2,max})^2} & q_3 &= \frac{\varepsilon_3}{(e_{3,max})^2} \\ r_1 &= \frac{\nu_1}{(u_{1,max})^2} & r_2 &= \frac{\nu_2}{(u_{2,max})^2} & r_3 &= \frac{\nu_3}{(u_{3,max})^2}, \end{aligned}$$

681 where, for all  $i \in \{1, 2, 3\}$ ,  $e_{i,max}$  and  $u_{i,max}$  denote the  
 682 maximum acceptable values of  $e_i$  and  $u_i$  (see Bryson's  
 683 rule [37]). Then, the tunable parameters  $(\varepsilon_1, \varepsilon_2, \varepsilon_3) \in \mathbb{R}_{>0}^3$   
 684 and  $(\nu_1, \nu_2, \nu_3) \in \mathbb{R}_{>0}^3$  are used to penalize, in order of  
 685 importance, the deviations of ACT, AO and  $P_{bank}$  with-  
 686 out unduly restricting the control effort. In the end, the  
 687 weighting matrices are given by:

$$688 Q = \text{diag}\left(\frac{30}{1.5^2}, \frac{20}{5^2}, \frac{15}{30^2}\right), R = \text{diag}\left(\frac{0.01}{1.25^2}, \frac{0.01}{1.25^2}, \frac{0.01}{0.04^2}\right).$$

689 The sampling time  $T_s = 60$  s and the prediction horizon  
 690  $N = 10$  are chosen so as to balance on-line complexity  
 691 and closed-loop stability. The NMPC algorithm is im-  
 692 plemented in MATLAB R2019a with CasADi v3.5.5 [38].  
 693 This open-source software provides several building blocks  
 694 which are especially useful for solving large-scale nonlinear  
 695 programming problems efficiently. Practically speaking, the  
 696 key asset of CasADi is its ability to quickly compute Jaco-  
 697 bians and exploit their sparsity pattern using algorithmic  
 698 differentiation and graph coloring techniques. Within this  
 699 framework, the reduced-order PWR model is simulated  
 700 with the stiff adaptive-step integration solver `IDAS` [26]  
 701 and the optimization problem is solved with the primal-  
 702 dual interior point solver `IPOPT` [39] using an Intel<sup>®</sup> Core<sup>™</sup>  
 703 i3-6100U processor with 16 GB of RAM. The closed-loop  
 704 interactions between the controller and the plant are rep-  
 705 resented by the sequential execution of both MATLAB<sup>®</sup>  
 706 and Simulink<sup>®</sup> files.

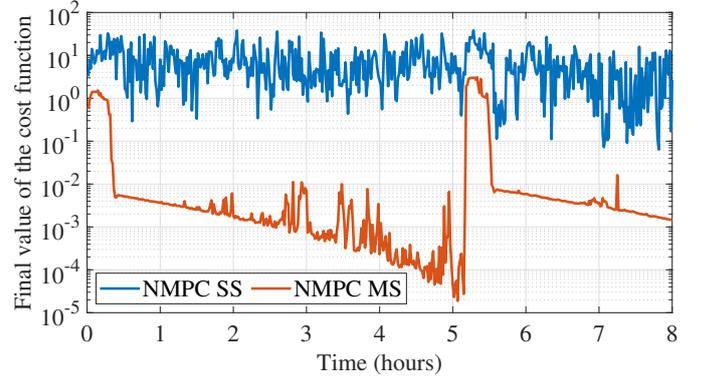


Fig. 4: Semi-log plot of the final value of the cost function (NMPC single-shooting vs NMPC multiple-shooting)

708 In order to choose between single-shooting and multiple-  
 709 shooting, both transcription methods were implemented and tested.  
 710 As can be seen in Fig. A.7 and A.9, the ACT deviation and the position  
 711 deviation of  $P_{bank}$  are almost identical for both methods. However,  
 712 the AO deviation, shown in Fig. A.8, is significantly smaller in  
 713 multiple-shooting because the behavior of  $H_{bank}$ , shown in  
 714 Fig. A.11, is less erratic than in single-shooting. The  
 715 same phenomenon can be observed, but to a lesser extent,  
 716 for  $P_{bank}$  in Fig. A.10 and for the boron concentration in  
 717 Fig. A.12. The reason behind this, as Fig. 4 demonstrates,  
 718 is that the optimization problem converges more quickly  
 719 to the solution when transcribed in multiple-shooting. In  
 720 fact, it turned out that further increasing the resolution  
 721 time in multiple-shooting had no effect on the final value  
 722 of the cost function. This suggests that the optimal solu-  
 723 tion can always be found within the allocated time, which  
 724 is why multiple-shooting is selected over single-shooting.  
 725

726 Overall, the performance of the NMPC controller is bet-  
 727 ter than that of the gain-scheduled controller. As can be  
 728 seen in Fig. A.13, A.14 and A.15, both controllers manage  
 729 to keep the ACT, the AO and the position of  $P_{bank}$   
 730 close to their reference signals. Yet, as can be observed in  
 731 Fig. A.16, A.17 and A.18, the gain-scheduled controller  
 732 tends to overuse the actuators in comparison with the  
 733 NMPC controller. This shows that the bandwidth of the  
 734 controller does not need to be particularly large even when

Table 1: Comparison between the gain-scheduled controller and the nonlinear model predictive controller

GSC	NMPC
Several linear models	One nonlinear model
Frequency domain design	Time domain design
Output-feedback control	State-feedback control
Robust design	Nominal design
Pre-computed gains	Online calculations
Continuous-time control	Discrete-time control
No anticipation	Preventive actions

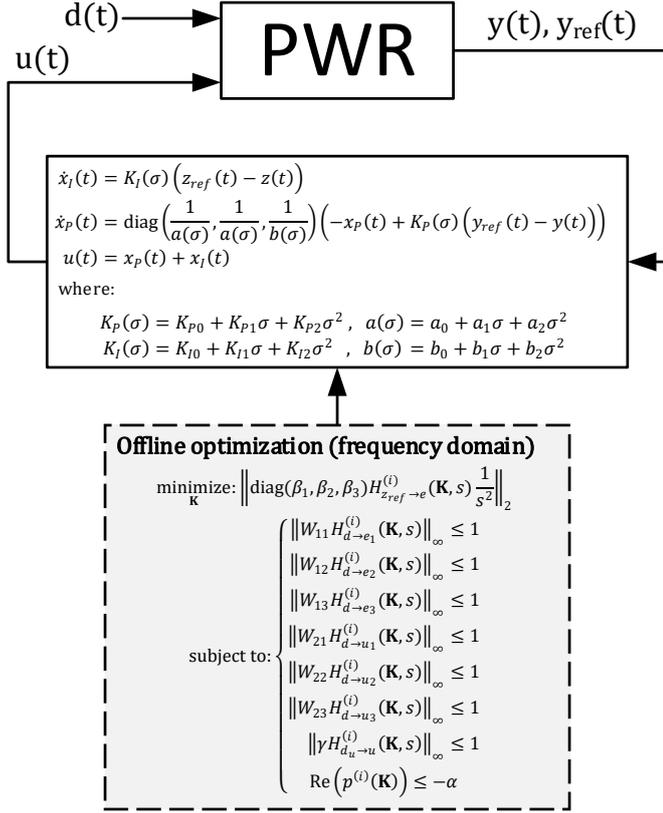


Fig. 5: Fixed-structure gain-scheduled control approach

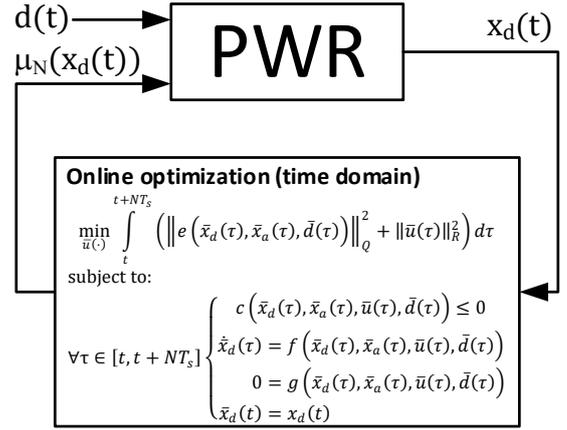


Fig. 6: Nonlinear model predictive control approach

Unfortunately, some state variables, such as the xenon concentration of the reactor core, cannot be physically measured. Thus, the NMPC controller cannot be practically implemented unless a nonlinear state observer is designed. This not only increases the overall complexity of the core control system but also raises additional concerns about its robustness.

## 6. Conclusion

In this paper, two advanced core control systems have been designed for flexible operation of future PWRs. The first one is a fixed-structure  $H_2/H_\infty$  gain-scheduled controller and the second one is an NMPC controller. The design methodologies of both controllers have been detailed and their performances have been compared on the intermediate complexity pressurized water reactor simulator of Framatome, named PWRSimu. The gain-scheduled controller is well adapted to cope with unexpected disturbances and model uncertainties. Moreover, its architecture is readable and easy to implement. However, some improvements should be made to avoid overusing the actuators (a dead-zone nonlinearity could be taken into account in the design of the controller using, e.g., the circle criterion). By contrast, the NMPC controller can handle a wide range of constraints and can easily deal with disturbances that are known in advance. Its predictive capabilities make it possible to achieve good performance while limiting the control effort. However, it is very difficult to analyze its robustness, especially when used together with a state-observer. In future work, the controllers will be combined in a hierarchical manner to overcome the limitations of both design strategies. The NMPC controller could be located at an upper level to provide a feedforward action to the plant as well as reference output trajectories to the gain-scheduled controller. Then, the gain-scheduled controller could be used at a lower level to reject unknown disturbances and to ensure that the closed-loop system stays close to the reference trajectory despite model uncertainties. This hierarchical scheme is also motivated by

frequency control is active. However, the NMPC controller may struggle to reject other sudden and unexpected disturbances, as the control inputs are only updated every minute. To give a concrete example, note that the principal effect of frequency control is to cause ACT fluctuations of about  $\pm 0.8$  °C. A closer look at Fig. A.13 reveals that these fluctuations cannot be properly mitigated by the NMPC controller because they are unpredictable and of relatively high frequency. The gain-scheduled controller, on the other hand, gives better results since it provides continuous feedback to the plant and was designed using robust control methods. As a matter of fact, robustness is still an open issue in the field of model predictive control [15]. Therefore, it should be mentioned that the reduced-order PWR model, which is embedded in the NMPC controller, derives from the one that is used for validation. Hence, it is not surprising that the NMPC controller performs better than the gain-scheduled controller, which was designed off-line based on simpler linear models. This is both a strength and a weakness of nonlinear model predictive control. While it is indeed better to use a very detailed and accurate model for prediction, it can also become an issue if the computation time of the NMPC control law gets too long. Worse, using an excessively complex model can even make the optimization problem intractable. Besides, it is also good to recall that the NMPC controller is a state-feedback controller whereas the gain-scheduled controller is an output-feedback one.

801 the multiple time scales of the plant. In fact, the upper  
 802 layer could anticipate future load variations and forecast  
 803 the evolution of xenon concentration while the lower layer  
 804 could simply focus on disturbance rejection and setpoint  
 805 tracking. It would also make sense to leave the **calcula-**  
 806 **tion** of boron concentration adjustments to the upper layer  
 807 since the **evolution** of **the** boron concentration is **substan-**  
 808 **tially** slower than the movements of the control rods.

809 **Conflict of interest**

810 None declared.

811 **Acknowledgment**

812 The authors would like to thank Framatome for the fi-  
 813 nancial support.

814 **Appendix A. Simulation Results**

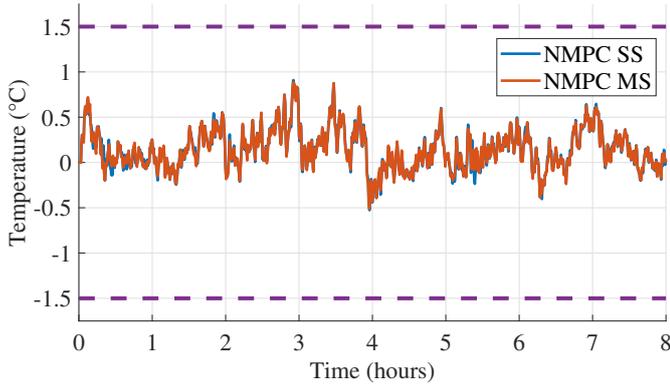


Fig. A.7: Average coolant temperature deviation (NMPC single-shooting vs NMPC multiple-shooting)

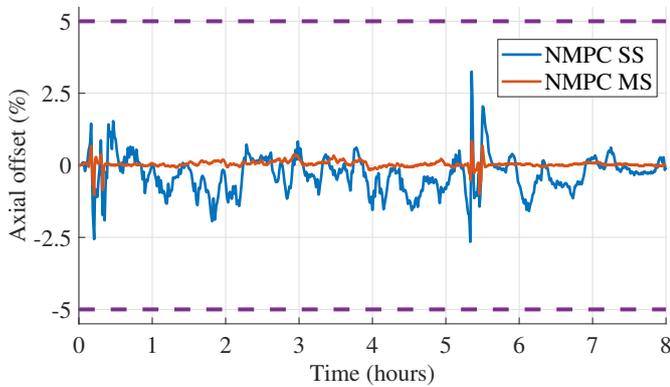


Fig. A.8: Axial offset deviation (NMPC single-shooting vs NMPC multiple-shooting)

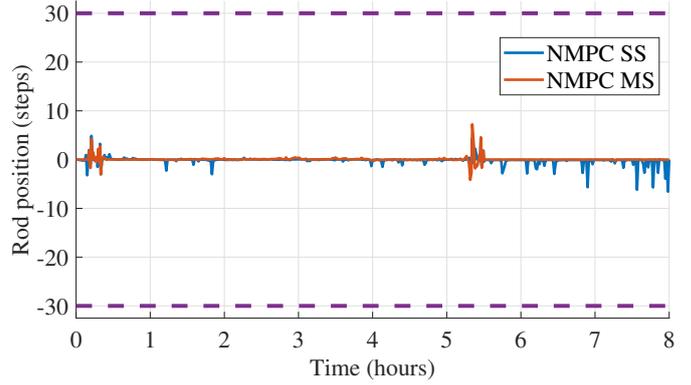


Fig. A.9: Position deviation of  $P_{bank}$  (NMPC single-shooting vs NMPC multiple-shooting)

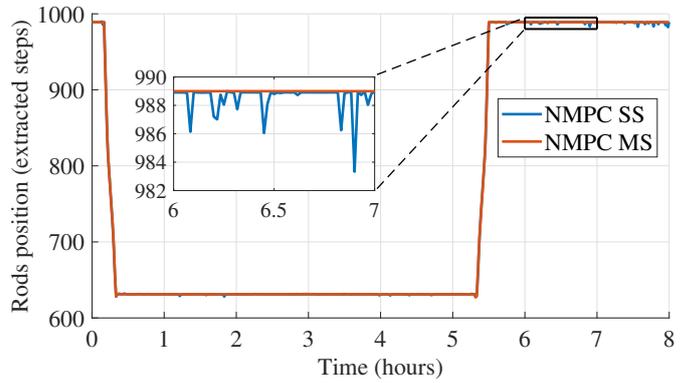


Fig. A.10: Position of  $P_{bank}$  (NMPC single-shooting vs NMPC multiple-shooting)

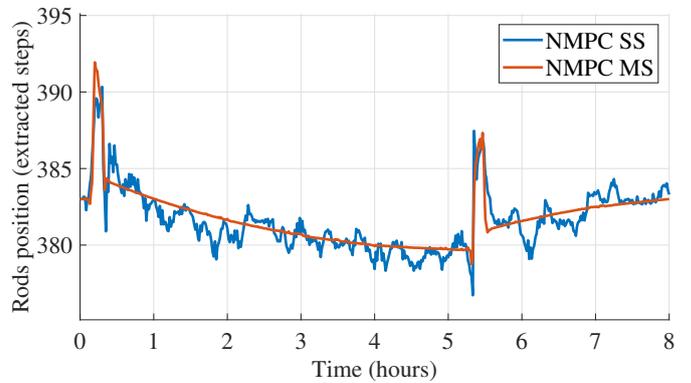


Fig. A.11: Position of  $H_{bank}$  (NMPC single-shooting vs NMPC multiple-shooting)

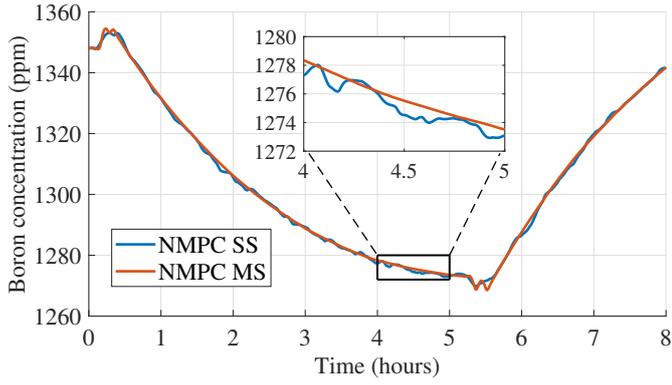


Fig. A.12: Boron concentration of the primary coolant (NMPC single-shooting vs NMPC multiple-shooting)

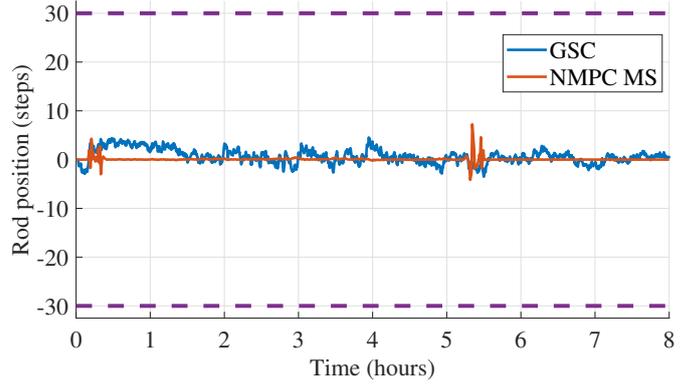


Fig. A.15: Position deviation of  $P_{\text{bank}}$  (Gain-Scheduled Controller vs NMPC multiple-shooting)

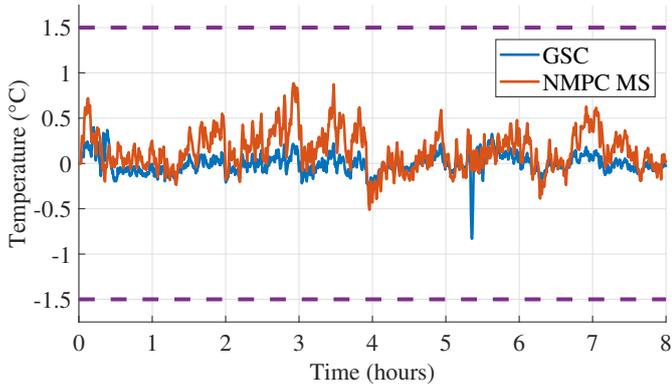


Fig. A.13: Average coolant temperature deviation (Gain-Scheduled Controller vs NMPC multiple-shooting)

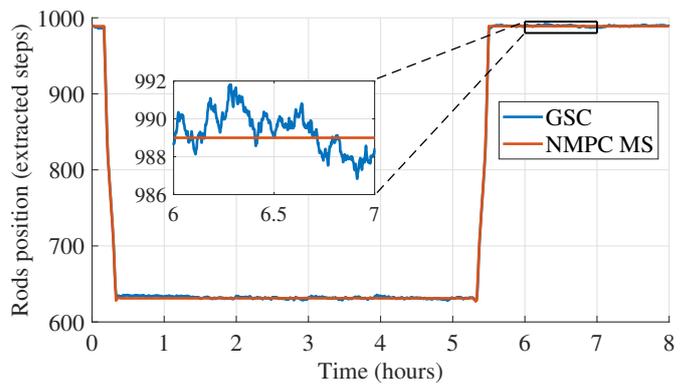


Fig. A.16: Position of  $P_{\text{bank}}$  (Gain-Scheduled Controller vs NMPC multiple-shooting)

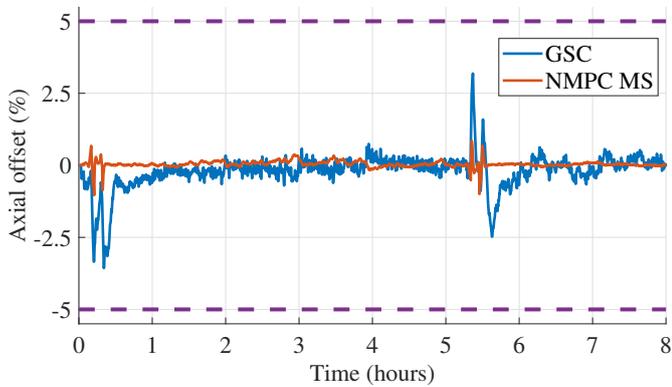


Fig. A.14: Axial offset deviation (Gain-Scheduled Controller vs NMPC multiple-shooting)

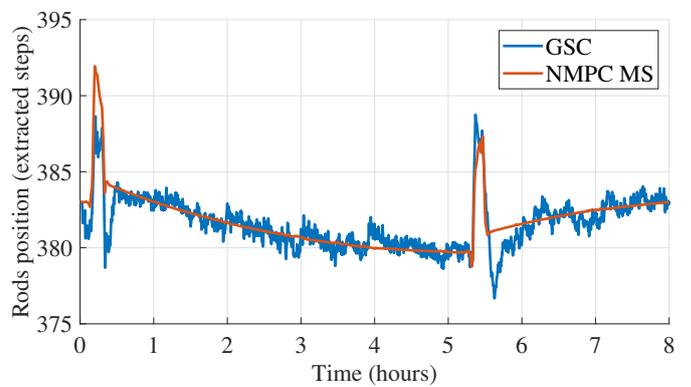


Fig. A.17: Position of  $H_{\text{bank}}$  (Gain-Scheduled Controller vs NMPC multiple-shooting)

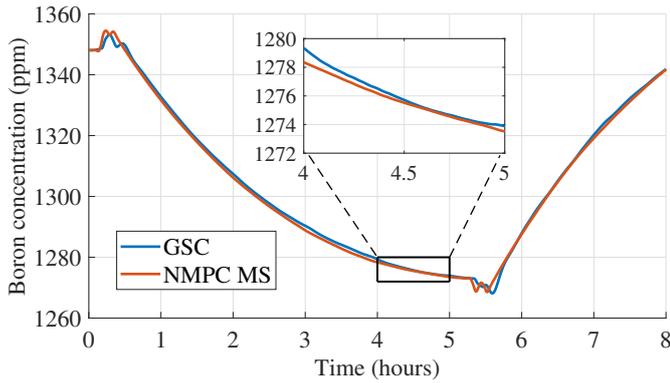


Fig. A.18: Boron concentration of the primary coolant (Gain-Scheduled Controller vs NMPC multiple-shooting)

## References

- [1] K. Kosowski, F. Diercks, Quo vadis, grid stability? Challenges increase as generation portfolio changes, *atw International Journal for Nuclear Power* 66 (2) (2021) 16–26.
- [2] P. Morilhat, S. Feutry, C. Lemaitre, J. M. Favennec, Nuclear power plant flexibility at EDF, *atw International Journal for Nuclear Power* 64 (3) (2019) 131–140.
- [3] IAEA, Non-baseload Operation in Nuclear Power Plants: Load Following and Frequency Control Modes of Flexible Operation, no. NP-T-3.23 in Nuclear Energy Series, IAEA Vienna, 2018.
- [4] J.-L. Mourlevat, Évolution des modes de pilotage, *Revue générale nucléaire* (3) (2007) 29–36.
- [5] A. Grossetête, ATMEA1 & EPR™ mode T core control innovative features for high operating flexibility, *Transactions of the American Nuclear Society* 111 (1) (2014) 1095–1098.
- [6] G. Li, X. Wang, B. Liang, X. Li, B. Zhang, Y. Zou, Modeling and control of nuclear reactor cores for electricity generation: A review of advanced technologies, *Renewable and Sustainable Energy Reviews* 60 (2016) 116–128.
- [7] L. Lemazurier, M. Yagoubi, P. Chevrel, A. Grossetête, Multi-objective  $H_2/H_\infty$  gain-scheduled nuclear core control design, *IFAC-PapersOnLine* 50 (1) (2017) 3256–3262.
- [8] L. Lemazurier, P. Chevrel, A. Grossetête, M. Yagoubi, An alternative to standard nuclear core control using a multi-objective approach, *Control Engineering Practice* 83 (2019) 98–107.
- [9] G. Dupré, A. Grossetête, P. Chevrel, M. Yagoubi, Enhanced flexibility of PWRs (mode A) using an efficient NMPC-based boration/dilution system, in: 2021 European Control Conference (ECC), IEEE, 2021, pp. 1092–1098.
- [10] M. G. Na, D. W. Jung, S. H. Shin, J. W. Jang, K. B. Lee, Y. J. Lee, A model predictive controller for load-following operation of PWR reactors, *IEEE Transactions on Nuclear Science* 52 (4) (2005) 1009–1020.
- [11] J. H. Kim, S. H. Park, M. G. Na, Design of a model predictive load-following controller by discrete optimization of control rod speed for PWRs, *Annals of Nuclear Energy* 71 (2014) 343–351.
- [12] H. Eliasi, M. Menhaj, H. Davilu, Robust nonlinear model predictive control for a PWR nuclear power plant, *Progress in Nuclear Energy* 54 (1) (2012) 177–185.
- [13] P. Gahinet, P. Apkarian, Automated tuning of gain-scheduled control systems, in: 52nd IEEE Conference on Decision and Control, IEEE, 2013, pp. 2740–2745.
- [14] D. Q. Mayne, J. B. Rawlings, C. V. Rao, P. O. Sokaert, Constrained model predictive control: Stability and optimality, *Automatica* 36 (6) (2000) 789–814.
- [15] D. Q. Mayne, Model predictive control: Recent developments and future promise, *Automatica* 50 (12) (2014) 2967–2986.
- [16] L. Lemazurier, Conception d’un système avancé de réacteur PWR flexible par les apports conjoints de l’ingénierie système et de l’automatique, Ph.D. thesis, Institut Mines-Télécom Atlantique (2018).
- [17] P. Reuss, Précis de neutronique, EDP sciences, 2012.
- [18] W. M. Stacey, Nuclear reactor physics, 3rd Edition, John Wiley & Sons, 2018.
- [19] D. J. Leith, W. E. Leithead, Survey of gain-scheduling analysis and design, *International journal of control* 73 (11) (2000) 1001–1025.
- [20] W. J. Rugh, J. S. Shamma, Research on gain scheduling, *Automatica* 36 (10) (2000) 1401–1425.
- [21] P. Apkarian, D. Noll, A. Rondepierre, Mixed  $H_2/H_\infty$  control via nonsmooth optimization, *SIAM Journal on Control and Optimization* 47 (3) (2008) 1516–1546.
- [22] M. Bouffier, L. Daudin, G. Rio, Science, an advanced 3D fuel management code package, in: TopFuel’97, Thomas Telford Publishing, 1997, pp. 5–30.
- [23] B. A. Francis, W. M. Wonham, The internal model principle of control theory, *Automatica* 12 (5) (1976) 457–465.
- [24] S. J. Qin, T. A. Badgwell, A survey of industrial model predictive control technology, *Control engineering practice* 11 (7) (2003) 733–764.
- [25] T. T. Hartley, G. O. Beale, S. P. Chikatelli, Digital simulation of dynamic systems: a control theory approach, Prentice-Hall, Inc., 1994.
- [26] A. C. Hindmarsh, P. N. Brown, K. E. Grant, S. L. Lee, R. Serban, D. E. Shumaker, C. S. Woodward, SUNDIALS: Suite of nonlinear and differential/algebraic equation solvers, *ACM Transactions on Mathematical Software (TOMS)* 31 (3) (2005) 363–396.
- [27] L. F. Shampine, M. W. Reichelt, J. A. Kierzenka, Solving index-1 DAEs in MATLAB and Simulink, *SIAM review* 41 (3) (1999) 538–552.
- [28] R. Findeisen, L. Imsland, F. Allgower, B. A. Foss, State and output feedback nonlinear model predictive control: An overview, *European journal of control* 9 (2-3) (2003) 190–206.
- [29] P. Kokotović, H. K. Khalil, J. O’reilly, Singular Perturbation Methods in Control: Analysis and Design, SIAM, 1999.
- [30] X. Chen, A. Ray, On singular perturbation of neutron point kinetics in the dynamic model of a PWR nuclear power plant, *Sci* 2 (2) (2020) 30.
- [31] L. Grüne, J. Pannek, Nonlinear model predictive control: Theory and Algorithms, 2nd Edition, Springer, 2017.
- [32] J. B. Rawlings, D. Q. Mayne, M. Diehl, Model predictive control: Theory, Computation, and Design, 2nd Edition, Nob Hill Publishing Madison, WI, 2019.
- [33] M. Diehl, H. J. Ferreau, N. Haverbeke, Efficient numerical methods for nonlinear MPC and moving horizon estimation, in: Nonlinear model predictive control, Springer, 2009, pp. 391–417.
- [34] L. Grüne, J. Pannek, Analysis of unconstrained NMPC schemes with incomplete optimization, *IFAC Proceedings Volumes* 43 (14) (2010) 238–243.
- [35] R. Findeisen, F. Allgöwer, Computational delay in nonlinear model predictive control, *IFAC Proceedings Volumes* 37 (1) (2004) 427–432.
- [36] R. Findeisen, L. Grüne, J. Pannek, P. Varutti, Robustness of prediction based delay compensation for nonlinear systems, *IFAC Proceedings Volumes* 44 (1) (2011) 203–208.
- [37] A. E. Bryson, Y.-C. Ho, Applied optimal control: optimization, estimation, and control, Routledge, 1975.
- [38] J. A. Andersson, J. Gillis, G. Horn, J. B. Rawlings, M. Diehl, CasADi: a software framework for nonlinear optimization and optimal control, *Mathematical Programming Computation* 11 (1) (2019) 1–36.
- [39] A. Wächter, L. T. Biegler, On the implementation of an interior-point filter line-search algorithm for large-scale nonlinear programming, *Mathematical programming* 106 (1) (2006) 25–57.