



DENOPI PROJECT DEVOTED TO SPENT FUEL POOL ACCIDENTS: OVERVIEW ON THE THERMAL HYDRAULICS EXPERIMENTAL FACILITIES



B. Migot, G. Brillant, J. Martin, S. Morin – I.R.S.N / Nuclear Safety Division, Safety Research
benoit.migot@irsn.fr

DENOPI PROJECT DEVOTED TO SPENT FUEL POOL ACCIDENTS: OVERVIEW ON THE THERMAL HYDRAULICS EXPERIMENTAL FACILITIES

Outline

- Introduction and context : spent fuel pool accidents
- Thermal hydraulics at the spent fuel pool scale: MIDI facility
- Thermal hydraulics at the fuel bundle scale: ASPIC facility
- Conclusions & next steps

DENOPI PROJECT DEVOTED TO SPENT FUEL POOL ACCIDENTS

Elements of context

- Fukushima Daiichi Unit 4 (March 11, 2011): 1331 spent fuels/2.26 MWth ⇒ Spent fuel pool loss of cooling triggered international efforts on SFP accidents

- At the light of the accident, active international works on SFPs :
 - OECD : Status report on SFP under loss of cooling and LOCA conditions (2015)
 - OECD : Phenomena Identification and Ranking Table on SFP-LOCA (2018)
 - European Community : stress tests program, benchmark of SFP transients with SA code
 -

- France : call for projects from the National Research Agency ⇒ DENOPI PROJECT (2014)



DENOPI PROJECT DEVOTED TO SPENT FUEL POOL ACCIDENTS

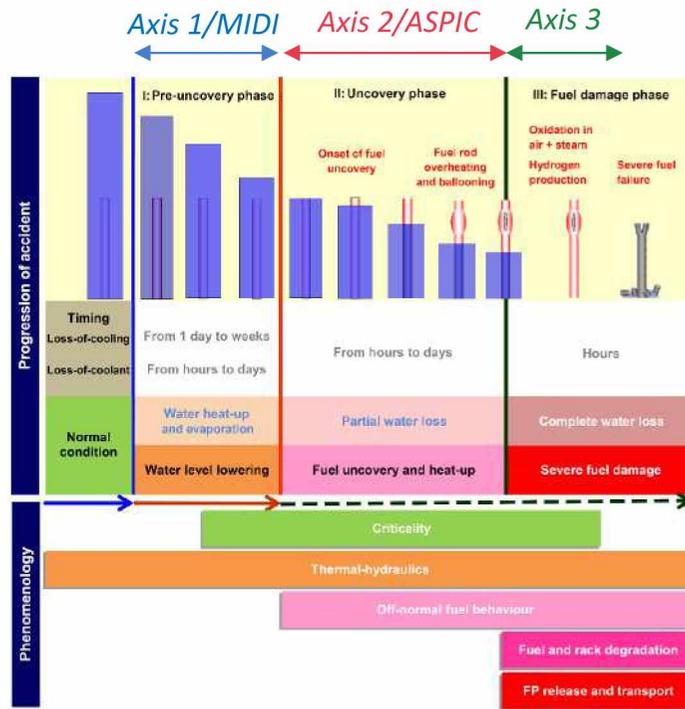
DENOPI project : overview

- Framework: post-Fukushima project, supported by the French government (ANR 11 – RSNR 006) – 2014/2022
- Participants: IRSN, French academic labs, national and international partners
- Organizer in 3 axis representing both different time frames of the accident and different scales of interest
 - Axis 1 at pool scale: before fuel uncovering ⇒ **MIDI facility**
 - Axis 2 at assembly scale: partial or total uncovering of fuel assembly ⇒ **ASPIC facility**
 - Axis 3 at fuel clad scale: clad degradation after uncovering



DENOPI PROJECT DEVOTED TO SPENT FUEL POOL ACCIDENTS

DENOPI project : overview

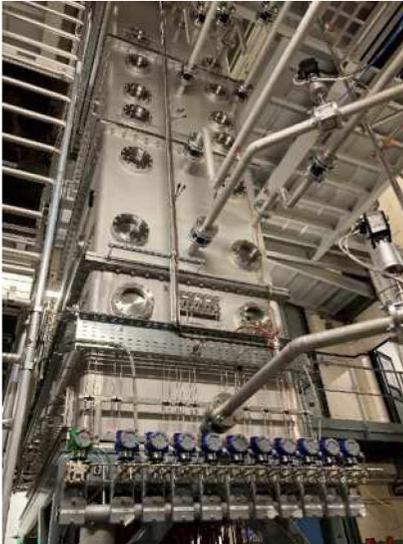


Temporal phases and phenomenology of SFP loss-of-cooling/coolant accidents*

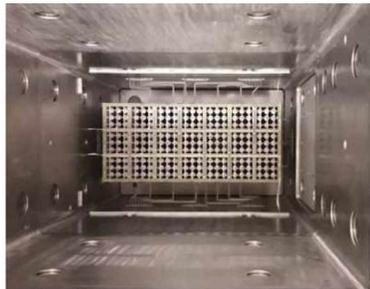


*figure extracts from the PIRT's report on SFP accidents

THERMAL HYDRAULICS AT THE SPENT FUEL POOL SCALE: MIDI facility



MIDI basin without the thermal insulation



Top view of the rack at the basin's bottom

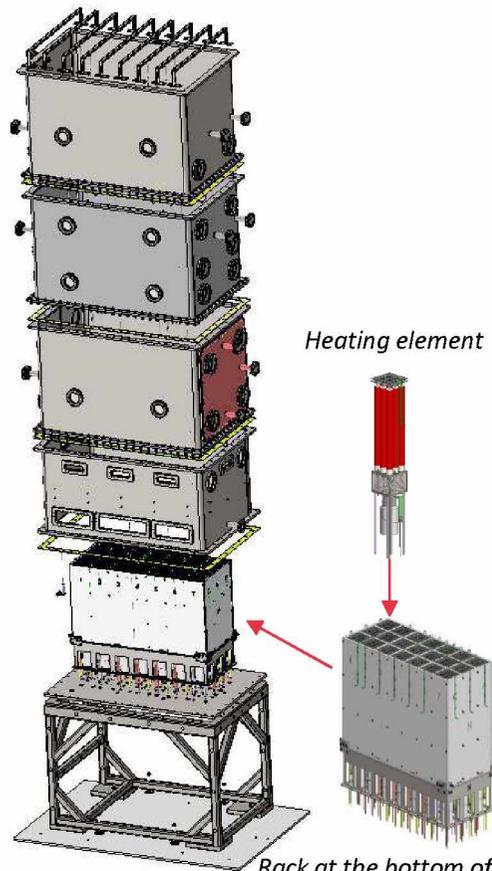
Main targets

- Get a better understanding and an experimental data base on the TH behavior of a SFP in case of a loss-of-cooling accident before FA uncover : study of coupled natural convection flows and vaporization regimes
- Contribution to the validation of models and computer codes
- Application to at-reactor SFPs, but also to away-from-reactor SFPs

Mock-up of a typical French 900 Mwe SFP at reduced scale (1/6th except for the pool height 1/3rd and heating power)

Parameters	MIDI facility	Typical PWR SFP	Scale ratio
Maximum water level (m)	4.0	12.0	1/3
Bundle active length (m)	0.7	~ 4.0	~ 1/3
Basin width (m)	1.42	8.50	1/6
Basin length (m)	2.10	12.60	1/6
Overall rack flow section (m ²)	0.65	22.86	~ 1/35
Free surface area (m ²)	2.982	107.1	~ 1/36
Bundle width (m)	0.225	0.225	1/1
Bundle porosity (-)	0.61	0.57	~ 1/1
Number of cells (-)	21 (7 X3)	792	~ 1/37
Power range (kW)	[50;300]	10 000	-
Max. power per bundle (kW)	50	58	~ 6/7
Volumetric power (kW/m ³)	[4.2;25.1]	7.8	-

THERMAL HYDRAULICS AT THE SPENT FUEL POOL SCALE: MIDI facility



■ Main characteristics

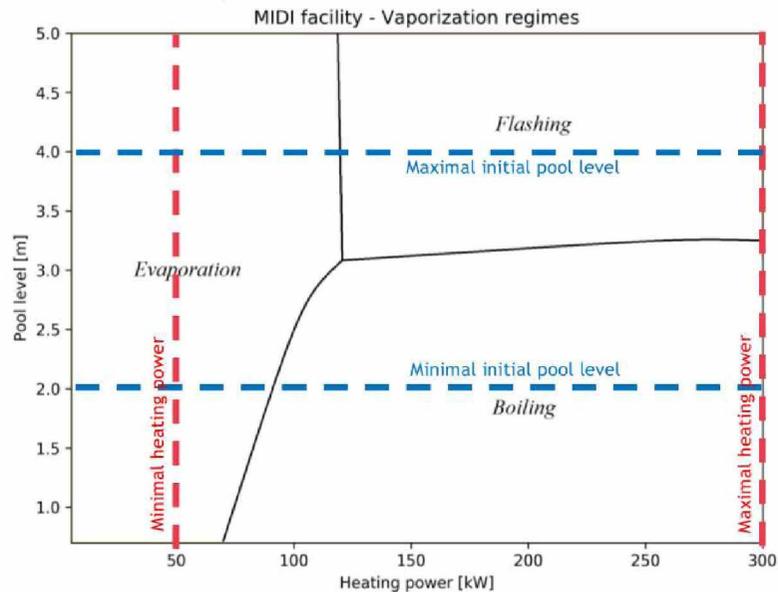
- Power of fuels simulated with electrically heated rods
 - 21 individual heating elements represent 21 cells of a storage rack - regular arrangement of the cells : 3X7
 - An heating element comprises an heated rods bundle (3X3)
- Control of the boundaries conditions
- Atmospheric volume above the free surface (suction hood extracting steam outside)
- Filling with demineralized water (until $\sim 12 \text{ m}^3$)

■ Main instrumentation

- rack thermal-hydraulic characterization (fluid temperatures, flowrates at each cell, heating power)
- void fraction at each outlet cell
- fluid temperatures in the volume above the rack
- pool levels (collapsed and swelled)
- water quality : conductivity & dissolved O₂
- gas temperature, absolute pressure, relative humidity of atmospheric volume above the free surface
- temperature of free surface
- temperature of lateral insulated walls

THERMAL HYDRAULICS AT THE SPENT FUEL POOL SCALE: MIDI facility

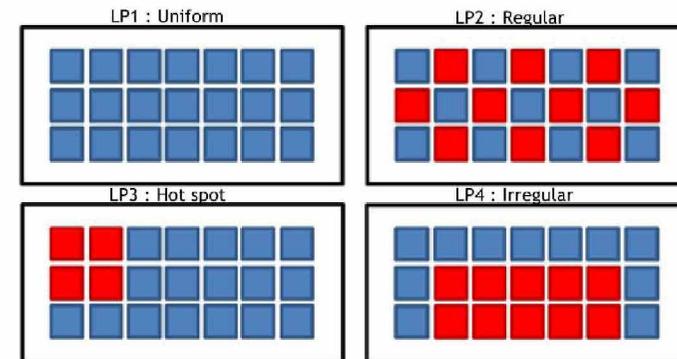
Test matrix based on 4 parameters



3 main vapor formation modes identified

- Evaporation at the free surface
- Nucleate boiling into the cells
- Flashing of superheated water in the upper part of the basin

- Pool level (1)
- Heating power (2)



➤ Loading pattern (3)

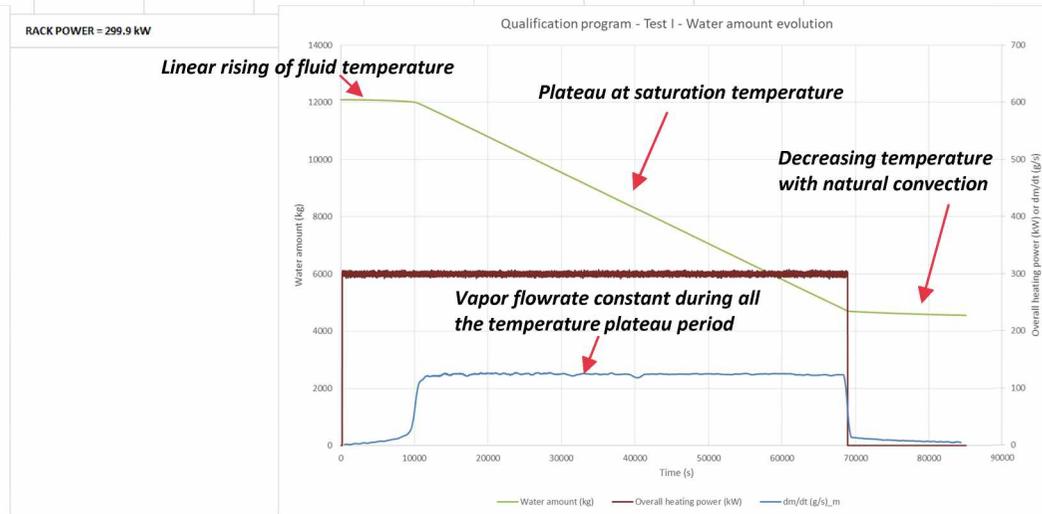
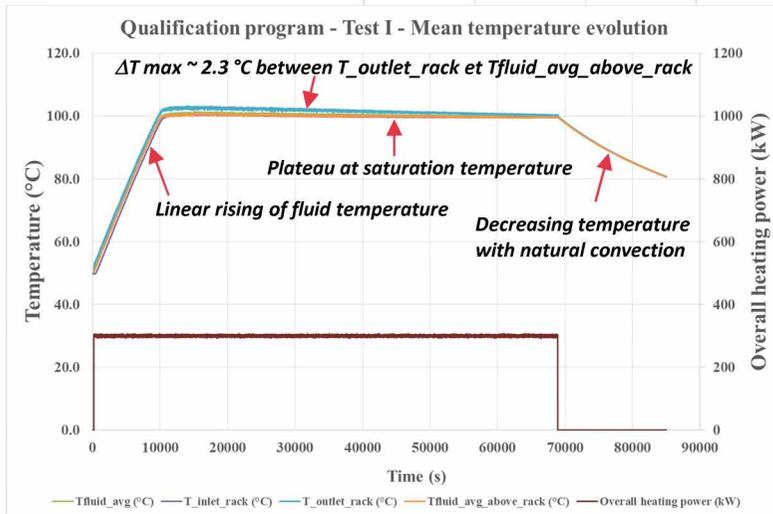
Reference	Type of loading pattern	"Hot cells"	"Cold cells"
LP1	"Uniform"	21 (100% overall power)	/
LP2	"Regular"	10 (91% overall power)	11 (9% overall power)
LP3	"Hot spot"	4 (91% overall power)	17 (9% overall power)
		4 (66% overall power)	17 (34% overall power)
LP4	"Irregular"	10 (91% overall power)	11 (9% overall power)

- Uniform loading pattern : ratio induced by the loading pattern
- 91% Hot -9% Cold : Management of SFP during shutdown in French 900 MWe : fuels under full-core discharged are moved into SFP
- Other ratios determined from an overall power of 300 kW and the maximum permissible power of a cell (50 kW)

➤ Ratio of heating power between "hot" and "cold" cells (4)

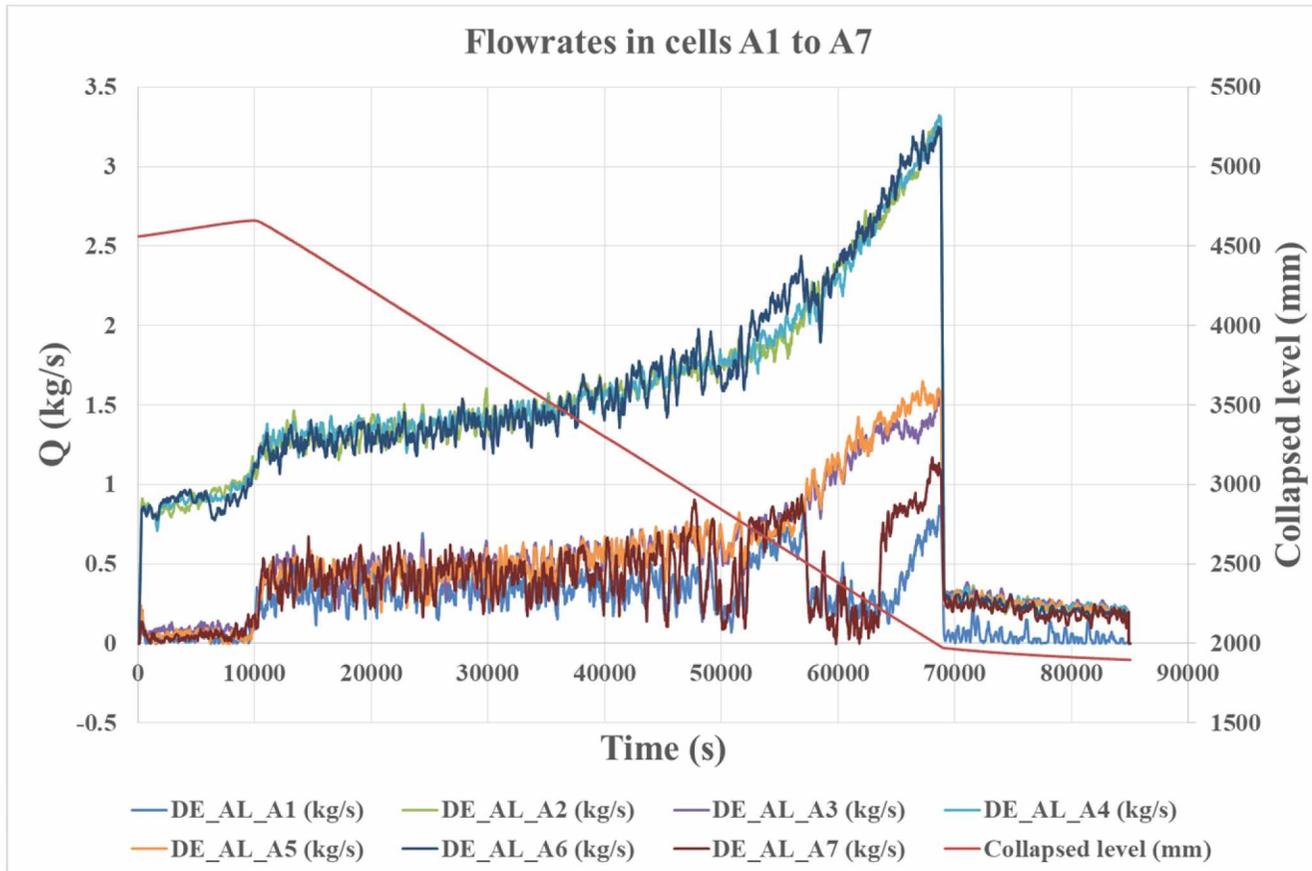
THERMAL HYDRAULICS AT THE SPENT FUEL POOL SCALE: MIDI facility

Some results from qualification tests



THERMAL HYDRAULICS AT THE SPENT FUEL POOL SCALE: MIDI facility

Some results from qualification tests



Main informations



- 1) Flowrates controlled by the power level in the cell
- 2) Flowrates evolutions seem showing also the different vaporization regimes
- 3) Increase of flowrates in the cells reflect the increase of the boiling in the cells

THERMAL HYDRAULICS AT THE FUEL BUNDLE SCALE: ASPIC facility



Insertion of the bundle in the cell



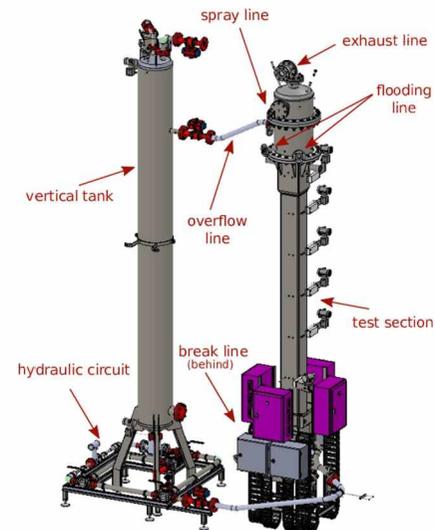
ASPIC facility in operation

Context

- Rod bundle stored in a spent fuel pool rack
- From boiling until the full uncover

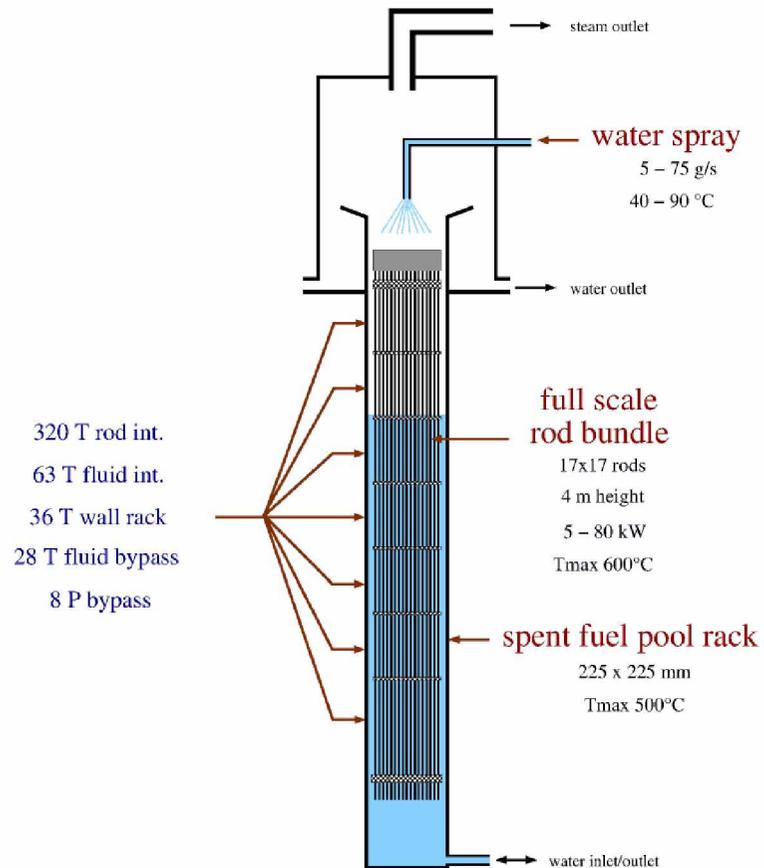
Main targets

- Assessment of the efficiency of a spray system to cool down a fuel assembly
- Analysis of the thermal exchanges during the bundle dewatering



Overview of the ASPIC facility

THERMAL HYDRAULICS AT THE FUEL BUNDLE SCALE: ASPIC facility



Main characteristics

- Full assembly scale, 17x17 electrically heated rods
- Geometric characteristics of a standard PWR assembly
- Power range of fuel bundle : 5 – 80 kW
- Size cell framing the fuel assembly: square tube 225 X 225
- Versatile configuration: no, partial or total uncovering of fuel bundle
- Water spray from the top: 0 – 75 g/s, 40 - 90°C
- Thermal insulation of all external surfaces

Main instrumentation

- 8 pressure measurements in the test section for void fraction estimation
- Wall temperatures of the rack
- Fluid temperatures inside the fuel bundle and in the gap between the rack/fuel bundle
- 80 heating rods instrumented with 4 thermocouples : 320 rods temperatures
- Water temperature and mass flowrate in the spray line
- Steam outlet flowrate

THERMAL HYDRAULICS AT THE FUEL BUNDLE SCALE: ASPIC facility

■ Test matrix based on :

- the comparison of tests carried out with/without the spray system
- test scenario with a stabilized water level

■ A0 - Boiling with a stabilized water level

- Initial state: hot water ($\sim 100^{\circ}\text{C}$) at a defined level
- Warm-up of the rod bundle
- Stop the warm-up close to the maximum bearable (temperature of the rods $\sim 500^{\circ}\text{C}$)



Assessment of spray efficiency

■ A1 - Spray cooling with a stabilized water level

- Initial state: hot water ($\sim 100^{\circ}\text{C}$) at a defined level
- Warm-up of the rod bundle
- Trigger of the spray on a rod temperature criterion (110°C)
- Stop the warm-up close to the maximum bearable (temperature of the rods $\sim 500^{\circ}\text{C}$)

THERMAL HYDRAULICS AT THE FUEL BUNDLE SCALE: ASPIC facility

- Test matrix based on :
 - a reference test and a parametric study around this reference test

■ Reference test and repeatability (scenario A) → 6 tests

- Bundle power: 20 kW
- Water height: 3 m
- Spray mass flowrate: 5 g/s
- Spray temperature: 40°C
- Rod temperature at the spray trigger: 110°C

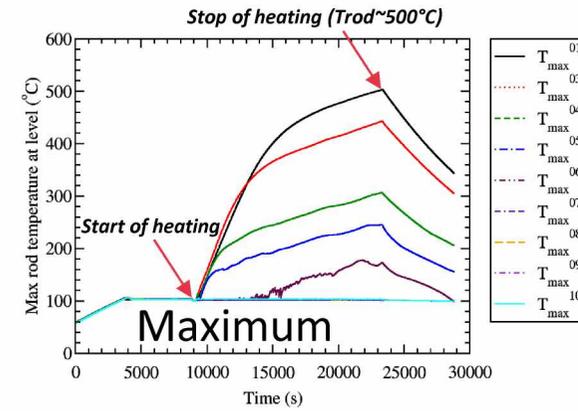
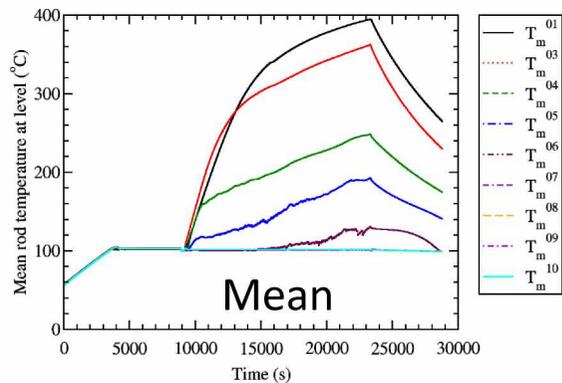
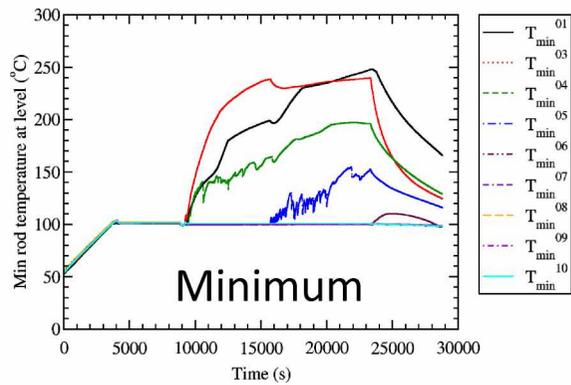


■ Variations around the reference test → 20 tests

- Bundle power: 10, 40 and 60 kW
- Water height: 0,1 and 2 m
- Spray mass flowrate: 10, 15 and 25 g/s
- Spray temperature: 80°C
- Rod temperature at the spray trigger: 200, 300 and 400°C

THERMAL HYDRAULICS AT THE FUEL BUNDLE SCALE: ASPIC facility

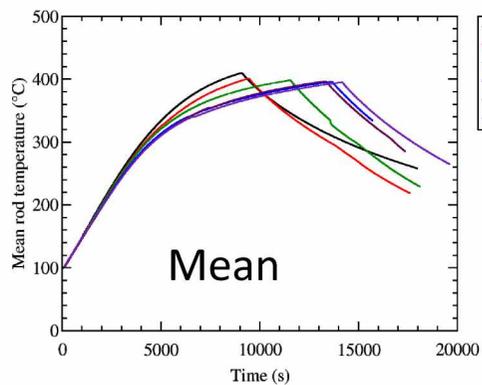
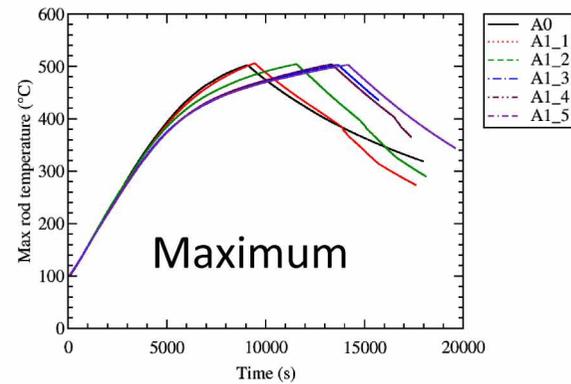
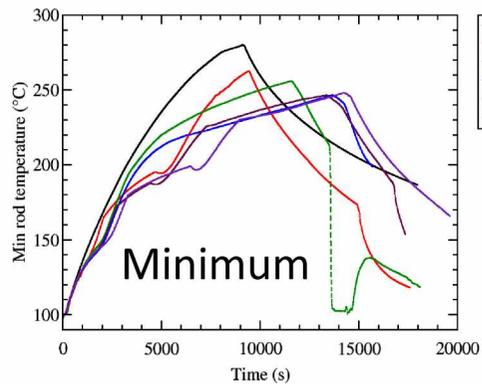
Typical results with test A1



- Spray trigger at 110°C
- Rods temperature statistics evolution with time at the upper levels

THERMAL HYDRAULICS AT THE FUEL BUNDLE SCALE: ASPIC facility

First results : comparison with/without spray (Tests A0/A1)



Time to reach 500°C is increased by about 50% with the spray (without taking into account A1_1 & A1_2 where water level not well stabilized)

Estimated temperature plateau

- Without the spray: 673 °C
- With the spray: 523 °C

CONCLUSIONS & NEXT STEPS

- Two new facilities, devoted to spent fuel pool accidents, were implemented in IRSN laboratories at Cadarache Center
 - MIDI: study of thermal hydraulic phenomena at pool scale before the fuel uncover stage
 - ASPIC: study of thermal hydraulic phenomena at fuel assembly scale before fuel degradation
- Experimentations in MIDI are now in progress, test report is expected by the end of 2022
- First experimentations in ASPIC are finished and first benchmark of the results with IRSN's DRACCAR code is in progress (tests without spray)
- End of DENOPI project is scheduled for the end of 2022, to date it is planned to continue the experiments and to propose a project as an OECD/NEA/CSNI Project for the 2023-2026 period.