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# Time-dependent reliability analysis of overpacks for high-level radioactive waste

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9 Abstract: In France, nuclear energy is extensively used for electricity production, and consequently 10 high-level radioactive waste are generated by the fleet of nuclear powerplants. This waste is planned to be disposed of in a deep geological repository. Engineers have only a limited feedback from similar 11 12 disposal facilities, as this project is innovative. This lack of knowledge and the timescales involved 13 induce uncertainties about the disposal environment and about the conditions of operation. Reliability 14 analysis is used here as it provides a suitable framework to deal with such uncertainties. This paper 15 focuses on the overpack, which isolates the waste from the environment during the early stage of 16 disposal (at least 500 years). The failure probability is investigated through a crack propagation criterion 17 using a Monte Carlo analysis performed on two levels. The approach is based on a dedicated stochastic 18 structural model including the corrosion rate, the mechanical loading and initial crack conditions. The 19 robustness of this approach is evaluated through sensitivity analysis, and by considering multiple 20 distributions for the variables elicited from expert's opinion.

21

#### 22 **1. Introduction**

23

24 About 10.6% of world electricity production (International Energy Agency 2017) and 70% of the French 25 electricity production in 2015 is generated by nuclear power plants. The management of the radioactive 26 waste resulting from this industry is an important issue. The European council directive 27 2011/70/EURATOM states that, at this time, deep geological repository is the safest option as the end 28 point of management of high-level radioactive waste. The methodology has been studied for several 29 decades, including laboratory tests, as well as the construction and operation of in-situ underground 30 research facilities, and is currently considered as the most feasible way of management of radioactive 31 waste (Chijimatsu et al. 2005; W. R.Alexander and Linda McKinley 2007; Rempe 2007; Lidskog and 32 Andersson 2002; Levy 2010). Deep geological repositories are investigated as an option for radioactive 33 waste management in many countries including Belgium, Canada, Czech Republic, Finland, France, 34 Germany, Hungary, Japan, Sweden, Switzerland and the USA (International Atomic Energy Agency 35 2001; Lidskog and Andersson 2002). However, since the waste and the nature of the ground of every 36 site are different, minimizing the risk of pollution requires facilities specifically engineered for each 37 project. The work presented in this paper falls within the framework of Cigéo which is the French project 38 of deep geological repository for Intermediate Level Long-Lived and High Level radioactive waste.

39 In this project high-level radioactive waste (HLW) is planned to be conditioned in non-alloy steel 40 overpacks and inserted in horizontal micro-tunnels. The main role of the overpack is to isolate the waste 41 from the environment long enough to significantly decrease its radiotoxicity and heat, which is estimated 42 to 500 years. Every overpack is inspected for manufacturing defects such as cracks, inclusions or

cavities. However, in operating conditions the overpack may display manufacturing defects smaller than
 the detection threshold, in addition to be subjected to corrosion and mechanical loading. Moreover, as

for every engineered system, this overpack is subjected to design and environmental uncertainties. These uncertainties are related to the extrapolation of phenomena over long time periods and limited amount of available data induced by the novelty of the project, making the analysis challenging. The uncertainties about the operating conditions induce uncertainties about the integrity of the system along the disposal time justifying the use of reliability methods (Hari Prasad et al. 2013; Kim et al. 2013; Bhargava, Mori, and Ghosh 2011).

7 The reliability methods have not been explored yet for the HLW facilities but they are already vastly 8 used for facilities with comparable constraints such as underground pipelines (Dundulis et al. 2016; 9 Amirat, Mohamed-Chateauneuf, and Chaoui 2006). These methods are based on the stochastic structural 10 approach, in which a mechanical model of the system is associated with uncertainties and used in a 11 reliability-based calculation procedure. It allows the estimation of the probability of occurrence of a 12 failure event (the failure probability) related to the response of the model (Lemaire 2013; Schuëller 1989). In this study the reliability method is coupled with a dedicated finite element model. Moreover, 13 14 as the failure event is rare, the numerical evaluation of the failure probability may involve a large number 15 of simulations. Hence, the issues in designing such a model are the computational time and the 16 parameterization to implement the variabilities (Sudret and Der Kiureghian 2002; Stefanou 2009; Patelli 17 et al. 2012). The reliability methods aim at reducing the number of numerical evaluations necessary to 18 estimate the probability of failure and increasing the accuracy of this estimation.

19 The failure event studied is the propagation of a pre-existing crack, it is associated with a fracture 20 mechanics criterion. For the sake of this analysis, a finite element (FE) model of the overpack including 21 the corrosion process is developed, allowing to consider the uncertain corrosion rates and mechanical 22 loadings. The time dependence is handled by carrying out independent static FE calculations at several 23 time steps until failure. A post-processing procedure is developed as well, allowing to simulate surface 24 cracks with uncertain positions and orientations from the results of the FE model. These features allow 25 us to analyse a large number of crack configurations with moderate numerical efforts. The reliability is 26 studied using a Monte-Carlo procedure based on two levels to maximise the number of simulations. The 27 sensitivity of the stress intensity factor is studied with respect to the input parameters as well as the 28 sensitivity of the failure probability with respect to the type of probability distribution. The interest of 29 this work is the deployment of structural reliability methods to a component of radioactive waste 30 repository problems.

The procedure of the analysis follows 3 steps: first a realisation of the random variables is generated; second, the finite element (FE) model is evaluated for several time steps until complete corrosion of the overpack; third, for each time step, the stress intensity factors associated with various randomly generated cracks are evaluated and compared to the critical stress intensity factor  $K_{1scc}$  to detect the failure.

The upcoming section of the paper presents an overview of the studied system and the main issues of the analysis. Then the stochastic structural model developed for the study is presented in details as well as the design features and the applied reliability method. The results of Monte-Carlo simulations and of sensitivity analyses are presented and discussed in section 3. The paper closes with the conclusions and outlook.

#### 41 2. Overpack operating conditions

42

43 Deep geological repositories are designed as the end point of management of radioactive wastes of high
44 and long-lived intermediate level activity. The disposal facilities are considered as multi-barrier systems
45 where geological and engineered barriers work together as Russian nesting dolls to contain and isolate
46 the waste from the biosphere. The design of the Cigéo project is based on an impermeable geological

47 clay rock formation (Callovo-Oxfordian claystone) able to contain the radioactivity over a long period

1 of time (ANDRA 2005). Underground facilities (access drifts, disposal cells) will be drilled in the 2 middle of the layer at 500m depth, one of them is shown in fig. 1.a. Such galleries are dedicated to 3 handling operations. HLW diposal cells are horizontal micro-tunnels of about 0.9 m in diameter. A non-4 alloy steel liner is inserted in the micro-tunnel, and a cement-based filling material is injected between 5 the liner and the rock for corrosion issues. HLW waste packages will be inserted inside the cell. They 6 consist of non-alloy steel overpacks, containing a stainless-steel primary package, inside of which the 7 vitrified waste is cast (fig. 1.b). A premature contact between the water and the primary package may 8 lead to an accelerated degradation of the glass matrix of the radioactive waste (Hoorelbeke 2009). 9 Therefore, the function of the overpack is to isolate the package from the environment long enough to 10 sufficiently decrease its radiotoxicity and heat. Moreover, one of the major goals of the Cigéo project is 11 the reversibility period. The principle is that for the first 100 years of operation, every waste package can be extracted at any time for limited cost. This requirement implies a mechanical stability of the steel 12

13 liner to allow the access to the waste for the first 100 years.

14

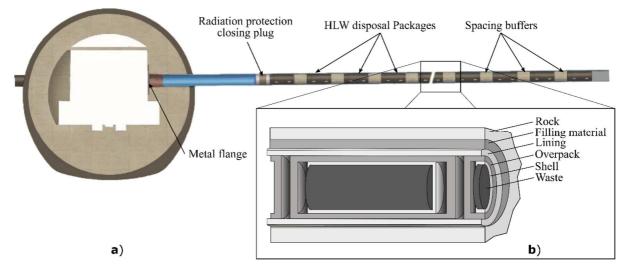


Figure 1. Overview of the HLW facilities as planned for the Cigéo project. a. General aspect of the HLW cell (courtesy of Andra).
 b. Russian nesting doll like barrier system

17 The first issue affecting the overpack in repository conditions is the corrosion. The argillaceous rock 18 contains water that may flow into the cells, this process is expected to progressively fill the HLW cell 19 and is represented by a time-dependant water level in the cell. The evolution of the water level is defined 20 by the following scenario. During the reversibility phase, a liquid water extraction system keeps the 21 tunnel dry. After this period the evolution of the water level results from the equilibrium of liquid and 22 gas phases in the cell, it has been studied by Croisé et al. (2011) for long-lived intermediate level waste 23 disposal cells in the project, i.e. considering the same rock layer. The results state that the rock saturation 24 increases and the water flows in. After a while, an equilibrium is reached between the water and gas 25 pressure and the water level stabilises to a constant value. This study focuses only on the case of a water 26 level stabilizing at an intermediate value filling half of the cell. The water induces two different corrosion 27 rates in the cell, one in water and one in wet atmosphere. The values and evolutions of these corrosion 28 rates are the results of chemical processes which are uncertain. The proposed evolution relies on in-situ 29 studies lead on the corrosion process of the overpack (Schlegel et al. 2014; Necib et al. 2017) and the 30 details about these uncertainties are given in section 3.2. Therefore, at each point of the overpack, the 31 corrosion rate is environment-dependent and the environment is time-dependent (related to the evolution 32 of the water level). Moreover, the corrosion rates in both environments are time-dependent and 33 uncertain. These processes induce a non-uniform corrosion of the overpack leading to a complex and 34 uncertain geometry of the corroded profile.

The second issue the overpack faces in operating conditions is related to mechanical loading. After the reversibility phase, the cells are closed and the access drifts backfilled so that the atmosphere in the cell

1 will progressively reach a pressure equilibrium with the surrounding rock, resulting in the overpack 2 being subjected to a uniform fluid pressure. Then, the corrosion process affecting the liner will 3 progressively reduce its effective thickness. This thickness reduction coupled with the rock pressure will 4 cause the buckling of the liner, which will come into contact with the overpack and transmit the rock 5 pressure. The buckling time is estimated by an empirical law established in earlier studies by Ngyuen 6 (2017). This model considers the buckling of an infinite confined cylinder subjected to an external 7 pressure. The contact area progressively grows as the corrosion reduces the thickness of the liner and 8 additional buckling occur. At long term (i.e. long after the total corrosion of the liner) contact area tends 9 to completely surround the overpack, the rock pressure is then directly applied to the overpack; this 10 loading is also uncertain. As a result, the mechanical loading is composed of a constant and deterministic 11 fluid pressure, and a time dependant (related to the evolution of the contact area) and uncertain contact

12 pressure.

13 The last issue taken into account in the model is a potential manufacturing flaw in the material. The 14 overpack is a welded assembly of 3 forged parts in low carbon steel (fig. 2). This material is chosen 15 because it has been widely used during the past decades (in civil engineering, in the pressure vessel 16 industry, etc.), it is easily weldable, its corrosion process is more predictable than stainless steel, in 17 addition to its good ductility properties. The top welding is more difficult to carry out because the package is already in the overpack. There are also constraints limiting the heat treatment necessary to 18 19 reduce the residual stress in the welding. Each overpack is inspected to detect possible flaws in the 20 material. It is then assumed that no flaw bigger than the detection threshold can remain undetected. 21 However smaller flaws are likely to remain in the in-situ overpacks, and their position and orientation 22 are uncertain. These flaws may have a critical effect on the overpack failure. In this study, surface flaws

are modelled by randomly generated semi-elliptical surface cracks that may initiate a fracture.



24

25 Figure 2. R7-T7 vitrified waste disposal package

26 The failure mode investigated in this paper is the fracture of the overpack due to the propagation of an 27 initial crack. The fracture occurs when the stress intensity factor exceeds a threshold value as described by the linear elastic fracture mechanics. Even if general corrosion is the main corrosion mode expected 28 29 for the overpack, brittle fracture has been studied to cover, in a safe design approach, the case of a 30 potential Stress Corrosion Cracking (SCC) or hydrogen embrittlement sensitivity of the steel in 31 repository conditions. Therefore, the failure criterion involves the SCC fracture toughness of the material K<sub>1scc</sub>, whose value is uncertain. The distribution affected to K<sub>1scc</sub> is discussed in section 3.2. It 32 33 relies on a study of the fracture toughness of the overpack under stress corrosion cracking conditions 34 (Necib et al. 2017), in which CT specimens have been loaded to 40 MPa $\sqrt{m}$  during 4000 hours

### **3 3.** Methods of analysis

4

5 This section discusses the applied strategy to implement the stochastic structural model of the overpack.
6 This implementation requires to identify the boundary conditions properly describing the in-situ
7 conditions, to build a dedicated FE model designed to be adaptable and robust enough to take into
8 account the uncertainties and the evolution of the geometry over time. The reliability analysis method
9 considered is a two levels Monte Carlo method based on the stochastic and time-dependent FE model.

10

#### 3.1. Evolution of the mechanical loading

exhibiting very limited propagation ( $<150 \mu m$ ).

11

12 As indicated in section 2, the mechanical loading is composed of a constant fluid pressure and a time-13 dependent and uncertain soil-liner pressure. The fluid pressure is defined in the FE model as a uniform 14 external pressure with the constant value of 5MPa corresponding to the in-situ pore pressure in the rock. 15 Defining the contact loading applied at the outer face of the overpack requires first to identify the 16 evolution of the contact area between the overpack and the buckled liner over time. The contact area is 17 composed of two diametrically opposed areas defined by a contact angle  $\alpha$  and representing the expected 18 buckling mode (fig. 3.a). The circumferential orientation of the contact areas is highly unpredictable 19 because it depends on local material and loading heterogeneities. Horizontal orientation has been 20 considered, to take into account the anisotropy of the excavation induced fracture network (Bumbieler 21 et al. 2015) which may result in a lower stiffness of the surrounding rock in this direction. As long as 22 the buckling of the liner has not occurred, the contact angle is equal to zero, and no corresponding load 23 is applied. The estimation of the buckling time using the law introduced in section 2 considers the rock 24 pressure, the initial thickness and the corrosion rate. The post-buckling behaviour and the expansion of 25 the contact areas, driven by the rock mechanical properties, is difficult to predict. It is then assumed, as 26 a first approximation, that  $\alpha$  increases linearly such that it reaches 180° after 3000 years. The pressure 27 profile on the contact area is parabolic such that the pressure is equal to zero at the edges of the contact 28 area and to its nominal value ( $P_c$ ) in the middle (fig. **3.b**). Hence, the expression of the contact applied 29 pressure is given by:  $P(\theta, t) = P_c \cdot h(\theta, t)$  with.

30 
$$h(\theta, t) = \begin{cases} \text{if } \alpha(t) < 180, \theta \in \left[\frac{-\alpha}{2}, \frac{\alpha}{2}\right], h(\theta, t) = 1 - \left(\frac{2\theta}{\alpha(t)}\right)^2 \\ \text{if } \alpha(t) \ge 180, \theta \in [-90, 90], h(\theta, t) = 1 - \left(\frac{2\theta}{\alpha(t)}\right)^2 \end{cases}$$

31 where  $P_c$  defines the amplitude of the applied pressure and carries all the uncertainties,  $h(\theta,t)$  is a function 32 of maximal value equal to 1, and carries the variation of pressure in time and space. The distribution of 33  $P_c$  is detailed in section 3.2. For  $\alpha$  greater than 180°, the evolution of the pressure profile is adapted so 34 it tends to a circular profile, representing the overpack directly subjected to an isotropic rock pressure.

35 It represents the isotropic in-situ stress field in the cross-section of disposal cells which are planned to

- 1 be drilled parallel to major principal stress  $\sigma_{\rm H}$  (Bumbieler et al. 2015).
- 2

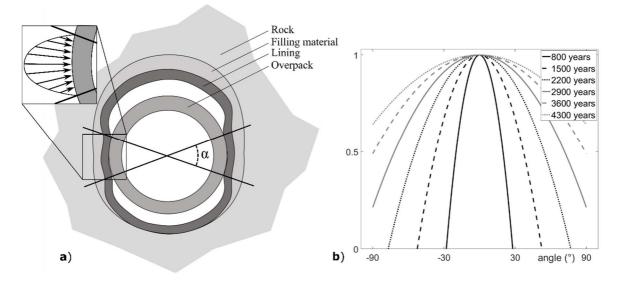


Figure 3.a. Schematic drawing of the contact angle b. Pressure profile for several time steps after buckling

3.2. Modeling of the uncertainties

6 As introduced in section 2, the water level is time-dependent according to the evolution presented in 7 figure 4.b. It induces two corrosion rates, CR<sub>a</sub> and CR<sub>w</sub> in the wet atmosphere and under water, 8 respectively. During the reversibility phase a possible inflow of oxygen from the access gallery is taken 9 into account and the corrosion rates are therefore considered steady for the first 100 years. After this 10 period, the corrosion rates decrease until reaching a new equilibrium state (fig. 4.a). The uncertainties 11 related to both corrosion rates may be associated with uncertainties about the kinetics of the chemical 12 processes due to uncertainties about environmental conditions evolution. Moreover, the involved 13 material and the environment are the same for both corrosion rates. As a result, if the corrosion rate in 14 the water is higher than the value predicted by the expert, the corrosion rate in the atmosphere is very 15 likely to be higher than the expert prediction as well. Therefore, the corrosion rates and their values over 16 time are considered as fully correlated and are affected by only one random variable C defined as a

1 multiplying factor applied to the evolution curves of both corrosion rates.

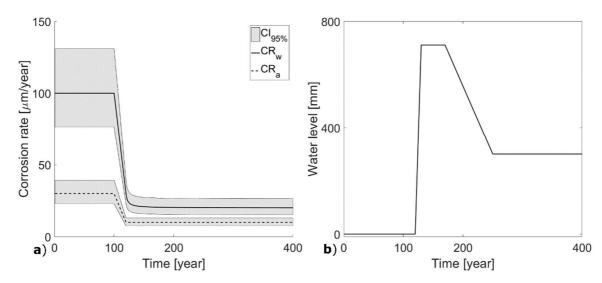


Figure 4.a. Evolution curve of the corrosion rates and the 95% confidence intervals associated to their probability density
 function b. Evolution curve of the water level

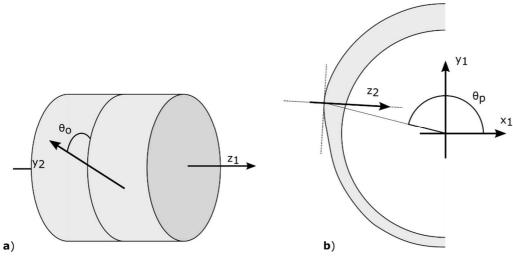
5 The distribution of C has been elicited from expert's judgment. The expert believes that the corrosion 6 rates have a nominal value c and that the credible interval that is the most likely to contain the actual value is  $\left[\frac{1}{1.5}c; 1.5c\right]$ . The lognormal distribution is chosen because a random variable following such a 7 distribution is equally likely to be greater than k and smaller than  $\frac{1}{k}$  which fits the expert's confidence 8 9 interval. By considering that the amplitude of the confidence interval equal to 6 times the standard 10 deviation, 99.7% of the realisations of C fall in the expert's interval. The contact pressure resulting from the transmission of the rock pressure to the overpack after buckling 11 is uncertain. The experts estimate the pressure to be most likely in the interval  $\left[\frac{1}{2}P_{c}; 2P_{c}\right]$  with  $P_{c}$  = 12

13 8MPa. The same strategy applied for corrosion is again used to define a log-normal distribution for the14 contact pressure.

15 The position and orientation of cracks in the overpack are characterized by two angles  $\theta_p$  and  $\theta_o$ 16 respectively (figs. **5.a**, **5.b**). Where  $\theta_p$  is the angle characterizing the position of the crack with respect 17 to the horizontal axis of the cross-section of the FE model and  $\theta_o$  is the angle between the major axis of 18 the crack and the circumferential axis of the FE model, it characterizes the orientation of the crack on 19 the surface of the overpack. As a first approximation, it is assumed that the crack does not exhibit 20 preferential position nor orientation. Therefore, uniform densities of range 180° are assigned to both  $\theta_p$ 

21 and  $\theta_0$ .

2





2 Figure **5.a.** Crack orientation angle, **b.** Crack position angle

As discussed in section 2 the fracture toughness of the overpack has been investigated in a study and
further studies are in progress but no statistical data are currently available. This lack of data prevents
us from identifying the distribution type and it is reasonably decided to model the fracture toughness
with a normal distribution with mean value of 40 MPa√m and coefficient of variation of 0.1

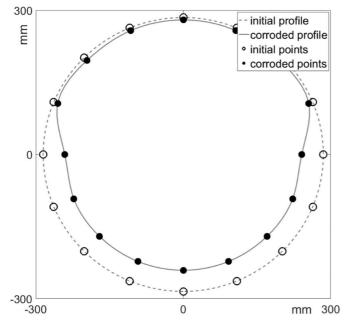
7

#### 8 *3.3. Evolution of the corroded profile*

9

The corrosion leads to a reduction of the overpack wall thickness. As a result, the cross-section of the
overpack is expressed in terms of time and corrosion rates, and its calculation requires a proper definition
of the corrosion rates and water level functions (see section 3.2).

13 The corroded thickness can be calculated at every time step and for every point of the cross-section, by 14 considering the water level and corrosion rate functions. In order to define the corroded geometry, the 15 corroded thickness is calculated at 16 points of the cross-section, and the geometry is defined by the 16 spline curve passing through these points (fig. 6). The spline curve is then used to define the cross-17 section of the FE model. As a first approximation the model is limited to the elastic behaviour of the 18 material to reduce the computational time. The FE model is composed of a tetrahedral quadratic mesh 19 to ensure that the mesh can be automatically generated for all the possible geometries of the corroded 20 overpack. The FE model is evaluated for time steps of 100 years until complete corrosion of the 21 overpack. The model is linear elastic, therefore it does not dependent on the loading history and 22 independent static simulations are carried out at each time step. The time step is chosen as to ensure



3 Figure 6 Geometry of the modelled cross-section of the corroded overpack

2

#### 5 *3.4. Simulation of the crack*

6

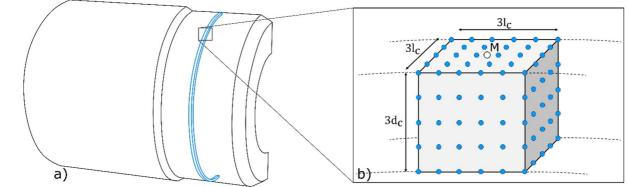
7 The failure mode investigated in this study is the propagation of a pre-existing crack. It is assumed that 8 the flaws introduced by the manufacturing process can be modelled as semi-elliptical cracks. The 9 dimensions of the crack are taken as the biggest acceptable flaw, according to quality control described 10 in the European standards (AFNOR 1998) regarding non-destructive testing of forged parts. Therefore, 11 the geometry of the crack is deterministic with both depth  $d_c$  and length  $l_c$  of 2mm (fig. 8); this value is 12 considered as upper bound for both dimensions. In order to reduce the computational time, the crack is 13 not directly modelled in the FE mesh, instead the crack is simulated by post-processing the results of 14 the uncracked model. As the reliability analyses implying a complex third-party model are generally 15 limited by the computational time (Patelli et al. 2012; Broggi and Schuëller 2011; Valdebenito and 16 Schuëller 2010; Goller et al. 2011), this strategy allows us to simulate numerous crack configurations 17 from a single evaluation of the FE model.

18 The stress intensity factor of the opening mode  $(K_1)$  is evaluated by Pommier et al. (1999) as a 19 generalization of the work of Newman and Raju (1981). The strategy used consists of modelling the 20 strain of un-cracked material and recovering the stress at the crack location. The stress intensity factor 21 is then deduced from the stress field. This approach is valid when the size of the crack is small compared 22 to the dimensions of the un-cracked ligament. The inputs of this model are the crack dimensions and a 23 space-dependent polynomial expression of the stress field associated with the opening mode of the 24 crack. The stress must be expressed in the coordinate system associated with the crack, and only the 25 stress orthogonal to the crack plan is necessary. The method is applicable only for semi-elliptical s cracks and gives good estimations of K1 for a wide range of crack sizes and stress fields. 26

Hence, a polynomial expression of the stress field of the un-cracked model around the crack position isrequired for the calculation of the stress intensity factor. The procedure consists of recovering the stress

- 29 tensor at several geometrical locations around the crack position. Then a polynomial expression of each
- 30 component of the tensor is fitted by least square regression. The most direct approach would be to

1 retrieve the data at several locations around the crack for each simulated crack, however the data 2 retrieving process can be time-consuming. Hence the data are instead recovered only once but at 3 locations distributed in all the area where the crack can be simulated. Only one data extraction is then 4 necessary to simulate all possible crack positions. Moreover, to reduce the data extraction time, the 5 possible crack locations are limited to the external surface of the overpack at one longitudinal position 6  $(z_c)$ . The conservatism of the model is ensured by choosing  $z_c$  to match the highest tensile stress concentration area. Therefore the stress tensor is retrieved at all points of a 3D grid centred on  $z_c$  and 7 8 composed of 700 circumferential points, five longitudinal points and five radial points. The grid width 9 and depth are  $3l_c$  and  $3d_c$  respectively (fig. 7). The detailed procedure of the regression and the rotation 10 in the local coordinate system associated with the crack, is as following:



11

Figure 7.a. Schematic representation of the grid of data recovering points b. Schematic representation of the slice of gird
 centred on the crack position used for the regression

Let  $B_1$  denotes the coordinate system associated with the overpack (i.e. the global coordinate system defined in the FE model) and  $B_2$  denotes the local coordinate system associated with the crack (defined by the orientation of the major and minor axes of the crack) as presented in fig. **8**, the transfer matrix expressing the change of coordinate system from  $B_1$  to  $B_2$  is expressed as:

$$18 \qquad M_{B_1}^{B_2} = \begin{pmatrix} -\sin(\theta_n)\sin(\theta_o) & \sin(\theta_n)\cos(\theta_o) & -\cos(\theta_n) \\ \cos(\theta_n)\sin(\theta_o) & -\cos(\theta_n)\cos(\theta_o) & -\sin(\theta_n) \\ -\cos(\theta_o) & -\sin(\theta_o) & 1 \end{pmatrix}$$
(1)

19 Let also  $\theta_n$  denote the angle between  $\overline{z_2}$  the major axis of the crack (fig. 8) and the horizontal axis of 20 the global coordinate system  $\overline{x_1}$  in the plan shown in (fig. 5.b). The stress tensors expressed in both 21 coordinate systems are denoted by  $\Sigma^{B_1}$  and  $\Sigma^{B_2}$  such that the term in the first row and the first column 22 of  $\Sigma^{B_1}$  is the tensile stress in the x-direction of the coordinate system  $B_1$ . The transfer relationship is 23 expressed as:

26 Figure 8. Geometry of the crack and associated coordinate system

For each realisation of the crack position (symbolized by the point M), a polynomial regression of the stress tensor is performed based on the stress data in the grid centred at the crack location, as shown in 1 fig.7.b. The polynomial expression is:

2 
$$\left[\sum^{B_1}\right] = [A] + [B]x_1 + [C]y_1 + [D]z_1$$
 (3)

3 where  $x_1$ ,  $y_1$  and  $z_1$  denote respectively the x, y and z coordinates in the coordinate system of the

4 overpack  $B_1$ . The four matrices A, B, C, and D are identified for each component of the stress tensor by

5 least square regression. Combining equations 2 and 3 results in the following expression:

$$6 \qquad \left[\Sigma^{B_2}\right] = \left[M_{B_1}^{B_2}\right] [A] \left[M_{B_1}^{B_2}\right]^t + \left[M_{B_1}^{B_2}\right] [B] \left[M_{B_1}^{B_2}\right]^t x_1 + \left[M_{B_1}^{B_2}\right] [C] \left[M_{B_1}^{B_2}\right]^t y_1 + \left[M_{B_1}^{B_2}\right] [D] \left[M_{B_1}^{B_2}\right]^t z_1 \qquad (4)$$

7 This expression gives the stress field in the crack coordinate system. The transfer from the coordinate  $B_2$  to the coordinated system  $B_1$  is defined as following:

9 
$$\begin{pmatrix} x_1 \\ y_1 \\ z_1 \end{pmatrix} = \begin{bmatrix} x_c \\ y_c \\ z_c \end{bmatrix} + \begin{bmatrix} M_{B_1}^{B_2} \end{bmatrix} \begin{bmatrix} x_2 \\ y_2 \\ z_2 \end{bmatrix}$$
 (5)

where x<sub>c</sub>, y<sub>c</sub>, z<sub>c</sub> and x<sub>2</sub>, y<sub>2</sub>, z<sub>2</sub> denote respectively the coordinates of the crack position expressed in the
coordinate system of the overpack B<sub>1</sub> and the coordinates in the local coordinate system of the crack B<sub>2</sub>.
By combining the Equations (4) and (5), the stress tensor expressed in the coordinate system B<sub>2</sub> can be
approximated by a polynomial expression with respect to coordinate system B<sub>2</sub>. The computation of K<sub>1</sub>

requires the coefficients of the polynomial stresses orthogonal to the crack plane (opening mode), i.e.

15 the term in the first row and first column of the stress tensor  $\sum^{B_2}$ .

As a result, the data are retrieved from the FE model only once per time step, all the crack simulations are performed from the data of the same grid and only linear regression has to be performed for each simulated crack position. As the data transfer between FE software and external post-processing software is time consuming, this strategy allows us to significantly reduce the computational time of the crack simulation procedure.

- 21 *3.5. Reliability Analysis*
- 22

The failure probability  $P_f$  is the integral of the probability density function on the failure domain. This integral can be estimated using a finite number of evaluations using the Monte-Carlo simulation method

25 (Lemaire 2013; Dunn and Shultis 2012).:

26 This approximation is valid for a sufficiently large number of simulations N, usually for N  $\geq$  100/P<sub>f</sub>. In 27 this study, random variables are divided into two categories depending on the numerical efforts 28 associated with the evaluation of the performance function. The random variables affecting the FE 29 model  $(C, P_c)$  are associated with considerable numerical efforts because, for each realization, the FE 30 analysis has to be performed for the whole lifetime. The random variables affecting only the crack ( $\theta_p$ ,  $\theta_o$ ,  $K_{1scc}$ ) are less demanding to evaluate because they only require post processing without 31 further evaluation of the FE model. Therefore, to allow for large number of simulations and to use the 32 33 advantage of the variables associated with moderate numerical effort, the Monte-Carlo simulations are 34 carried out on two levels with two different numbers of simulations. This method was first investigated 35 by Weitz and al. (2016) as an improvement of the Monte-Carlo method in such cases. The random 36 variable set can be divided into two sets  $\mathbf{X} = (\mathbf{X}_1, \mathbf{X}_2)$ , where  $\mathbf{X}_1 = (C, P_c)$  and  $\mathbf{X}_2 = (\theta_p, \theta_o, K_{1scc})$ . 37 Equation can be written as:

38 
$$P_{f} = \int_{-\infty}^{+\infty} I(\mathbf{x}) f(\mathbf{x}) d\mathbf{x} = \iint_{-\infty}^{+\infty} I(\mathbf{x}_{1}, \mathbf{x}_{2}) f_{1}(\mathbf{x}_{1}) f_{2}(\mathbf{x}_{2}) d\mathbf{x}_{1} d\mathbf{x}_{2}$$
(6)

where *I* is the classification function returning 1 in the failure domain and 0 otherwise. The Monte-Carlo
 simulation is applied twice in order to transform both integrals into sums.

$$3 \qquad P_{f} \approx \int_{-\infty}^{+\infty} \frac{1}{N_{2}} \sum_{k=1}^{N_{2}} I(\mathbf{x}_{1}, \mathbf{x}_{2}^{(k)}) d\mathbf{x}_{1} \quad \approx \frac{1}{N_{1}N_{2}} \sum_{j=1}^{N_{1}} \left( \sum_{k=1}^{N_{2}} I(\mathbf{x}_{1}^{(j)}, \mathbf{x}_{2}^{(k)}) \right)$$
(7)

4 With this strategy  $N_1.N_2$  evaluations of I are available for each time step. They are coming from  $N_1$ 5 evaluations on the FE model and, for each of them, evaluating the stress intensity factor resulting from 6  $N_2$  simulated cracks. It allows us to choose  $N_1$  and  $N_2$  according to the respective computation time of 7 the FE model and the post-processing, instead of being limited by the most time-consuming. In this 8 study, 500 simulations of the variables related to the finite element model ( $N_1$ ) are evaluated and 100,000 9 cracks ( $N_2$ ) are simulated for each of them and for each time step (t) until complete corrosion of the 10 overpack. Therefore, the failure probability is expressed as follows:

11 
$$P_{f}(t) \approx \frac{1}{N_{1}N_{2}} \sum_{k=1}^{N_{2}} \sum_{j=1}^{N_{1}} I(\theta_{p}^{(k)}, \theta_{o}^{(k)}, K_{1scc}^{(k)}, C^{(j)}, P_{c}^{(j)}, t)$$
 (8)

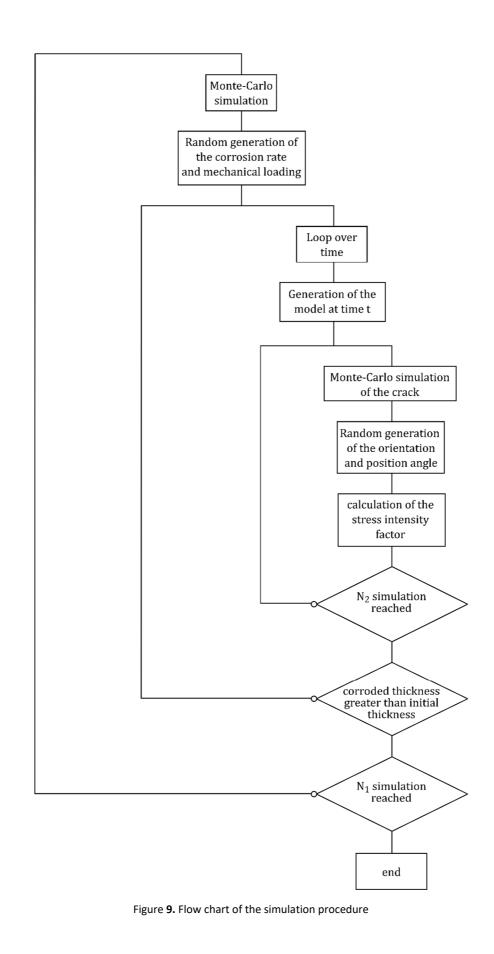
12 where *I* is associated to the performance function *g*:

13 
$$g(\mathbf{x}_{1}^{(j)}, \mathbf{x}_{2}^{(k)}, t) = K_{1scc}^{(k)} - K_{1}(\theta_{p}^{(k)}, \theta_{o}^{(k)}, C^{(j)}, P_{c}^{(j)}, t)$$
 (9)

hen full corrosion is reached (i.e. corrosion of all the wall thickness of the overpack), the water can be
in contact with the primary package which defines the failure of the system. Therefore, the failure occurs
either by propagation of the initial crack or by complete corrosion of the overpack thickness. The

17 corrosion state is checked by comparing the maximal corroded thickness to the initial thickness. The

18 simulation procedure is detailed in the flow chart presented in figure 9.





4 The computing time issue of the Monte-Carlo method has also motivated the development of other

methods known as the advanced Monte-Carlo methods such as Subset simulations (Au and Patelli 2016) and Line Sampling (M. de Angelis, Patelli, and Beer 2015). Unfortunately, these methods are not designed for using on a two-level Monte-Carlo and cannot be directly applied in such cases. The twolevel approach has been chosen over the advanced Monte-Carlo methods because of the strong difference in the computational effort of the two levels and the flexibility it offers in the computational time management

7

#### 8 *3.6. Sensitivity of the result regarding the input parameters*

9

An important outcome of this study is to identify the parameters with large influence on the failure
probability. This knowledge allows us to focus engineering efforts on the most important variables to
improve the design of the system and more generally to get a better understanding of the studied system.
This is the purpose of sensitivity analysis, the applied strategy here consists in identifying the input
parameters with large influence on the model output (the value of the stress intensity factor).

15 The input-output relationship is studied with a scatterplot method (Hamby 1994). The idea is first to 16 perform Monte-Carlo simulations and then to display the output as a function of the value of only one 17 of the inputs. If the output is significantly dependent of the considered input and if enough realisations 18 are available, the scatterplot reveals a relationship pattern and its sharpness gives information about the 19 strength of the relationship. As the problem is time-dependent, the sensitivity analysis can be performed 20 at each time step. The simulations are also performed on two levels implying two different numbers of 21 simulations. The sensitivity analysis is performed separately on each level and in order to free one 22 analysis from the other level, the considered values are:

$$\mathbf{K}_{1}(\mathbf{x}_{1}) = \int_{-\infty}^{+\infty} \mathbf{K}_{1}(\mathbf{x}_{1}, \mathbf{x}_{2}) f(\mathbf{x}_{2}) d\mathbf{x}_{2}$$

24

23

25 
$$K_{1}(\mathbf{x}_{2}) = \int_{-\infty}^{+\infty} K_{1}(\mathbf{x}_{1}, \mathbf{x}_{2}) f(\mathbf{x}_{1}) d\mathbf{x}_{1}$$

#### 26 *3.7. Influence of the choice of the distribution on the failure probability*

27

28 The considered distribution for the random variables are based on expert's opinion which is inaccurate 29 unless when based on several opinions (Ayyub 2001; O'Hagan 2006). Therefore, the real distributions 30 of the parameters may differ from the fitted ones. Hence, the influence of such an event on the validity 31 of the conclusions regarding the reliability of the overpack is investigated. For the sake of this analysis 32 five alternative distributions are considered for the variables associated with high computational effort; 33 i.e. the contact pressure P<sub>c</sub> and the corrosion rate C. Two alternative lognormal distributions are 34 considered with the same mode as the reference distribution, but the interval provided by the expert is 35 now assumed to fit five or seven times the standard deviation. Three additional distributions are tested 36 and chosen to be bounded distribution (triangular, log-triangular and beta distributions) with bounds 37 fitting the confidence interval and the mode equal to the nominal value given by the expert. (The  $\alpha$ 38 parameter of the beta distribution is chosen arbitrarily to be equal to two).

39 The approach of this study is to change the density of the studied random variables and to draw the new 40 evolution curve of the failure probability. The effect is then studied graphically without using dedicated 41 sensibility indicators. The most direct approach would be to conduct Monte-Carlo simulations using the 1 alternative distributions. However the computational efforts would be significant. Therefore a different

approach is used; it is based on a reweighting strategy which may be compared to importance sampling
 (Rubinstein and Kroese 2008). The main idea is to estimate something about a distribution using

4 observations form a different distribution (Sergienko et al. 2012; Papaioannou, Breitung and Straub

5 2018; Hesterberg 1996, de Angelis, Patelli, and Beer 2013). As introduced in section 3.5 if the random

6 variable **X** follows an alternative distribution  $\overline{f}$ , the alternative failure probability is expressed as:

7 
$$P_{\bar{f}} = \int_{-\infty}^{+\infty} I(\mathbf{X}) \bar{f}(\mathbf{X}) d\mathbf{X}$$

8 Then the numerator and denominator of the expression can be multiplied by the reference density f:

9 
$$P_{\bar{f}} = \int_{-\infty}^{+\infty} I(\mathbf{X}) \frac{f(\mathbf{X})}{f(\mathbf{X})} \bar{f}(\mathbf{X}) d\mathbf{X} = \int_{-\infty}^{+\infty} I(\mathbf{X}) \frac{\bar{f}(\mathbf{X})}{f(\mathbf{X})} f(\mathbf{X}) d\mathbf{X}$$
(10)

10 Then the Monte-Carlo principle can be applied and the integral is turned into a sum of samples  $X_f^{(j)}$ 11 generated following the reference density f:

12 
$$P_{\bar{f}} \approx \frac{1}{N} \sum_{j=1}^{N} I(\mathbf{x}_{f}^{(j)}) \frac{\bar{f}(\mathbf{x}_{f}^{(j)})}{f(\mathbf{x}_{f}^{(j)})}$$
 (11)

13 This expression denotes that the alternative failure probability can be estimated with weighted 14 realisations of **X** following the reference distribution f. The main benefit of this approach is that the 15 sample of realisations of the random variables following the reference density and their output  $I(\mathbf{X}_{f}^{(j)})$ 16 is already available from the Monte-Carlo simulation. Therefore the alternative failure probability can 17 be estimated without further evaluation of the FE model. Comparing the estimated failure probability 18 with the reference one give information about the influence of changes in the distribution of each random 19 variable. The goal is to assess the robustness of the model without performing additional simulations.

20

#### 21 4. Results

22

#### 23 *4.1. Failure probability*

24

25 The failure probability over time is estimated with the two level Monte Carlo method presented in 26 section 3.5. The FE model is evaluated using 500 realisations of the pressure contact (P<sub>c</sub>) and the 27 corrosion rate (C) for time steps of 100 years until complete corrosion of the model. At each time step 28 an independent linear elastic calculation is carried out considering the relevant corroded geometry and 29 mechanical loading. For each evaluation and at each time step the stress intensity factor is estimated for 30 simulated cracks resulting from 100.000 realisations of the position angle ( $\theta_n$ ) and the orientation angle  $(\theta_0)$ . Finally for each evaluation the values of the stress intensity factor (K<sub>1</sub>) are compared to 100.000 31 32 realisations of the critical stress intensity factor ( $K_{1scc}$ ).

33 The simulations are carried out sequentially with Abaqus and the evaluation of one time series (one

34 realisation of  $P_c$  and C) takes about 2h on an Intel Core i7-6700 CPU with 32 Go of memory. The

35 100.000 post-processing calculations associated with the time series take about the same time (2h). The

36 FE calculations and the post-processing can be carried out independently and simultaneously.

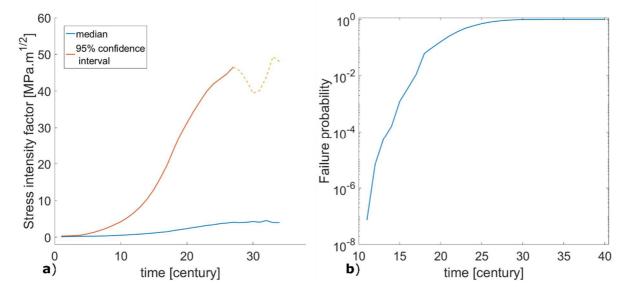
In order to increase the accuracy of the estimation for the small probabilities, 500 additional evaluationof the FE model and for each 200.000 crack simulations are performed for 8 time steps between 1000

- and 1700 years. This period corresponds to the first failures observed, therefore increasing the size of
   the evaluated sample allows us to estimate the failure probability more accurately.
- 3 The resulting stress intensity factors (K<sub>1</sub>) are represented in fig.10.a in terms of median value and upper 4 bound of the 95% confidence interval over time. A negative stress intensity factor value is obtained 5 when the equations proposed by Pommier et al. (1999) are applied to a crack subject to a compressive 6 loading. These values are considered equal to zero in the processing of the results which does not affect 7 the median curve nor the estimation of the failure probability  $(P_f)$ . The curves prove that the dispersion 8 of the stress intensity factor increases with time while the median value increases very slowly. It is 9 observed that cracks are almost equally likely to be subject to a compressive or tensile loading. Therefore, the median value of the stress intensity factor remains close to zero. However, with time 10 11 increases, the mechanical loading increases and the resulting stress in the material increases as well. As 12 a result, the scatter of the stress intensity factor increases with time (and in turns the upper quantiles of 13 the stress intensity factor are increased). The latest part of the confidence interval curve exhibits irregular 14 shape, this result comes from the decreasing number of realisations available to estimate the values of 15 interest. The reason is that once the FE model is completely corroded it cannot be evaluated anymore.
- 16 Therefore, with time increasing, more and more realisations among the 500 FE evaluations are lacking
- 17 in the results of the  $K_1$  values until no value remains available at 3500 years. This constraint causes the
- 95% confidence interval and the median value to be evaluated with decreasing accuracy along the time,affecting the shape of the curve at late stages.

20 The figure **10.b** shows the failure probability over time. No failure is observed until 1100 years where

the failure probability is  $7.3 \times 10^{-8}$  and P<sub>f</sub> is equal to one at 3500 years. The failure probability at 500

22 years (i.e. the specified lifetime of the overpack) is lower than  $7.3 \times 10^{-8}$  no failure occurred at this time.



23

Figure 10.a. Median value and upperbound of the 95% confidence interval of the  $K_1$  values over time. **b.** Estimated failure probability over time in semi logarithmic scale.

26 4.2. Parameter sensitivities

27

The sensitivity analysis is conducted with scatterplot analysis as detailed in section 3.6. The values are studied for two time steps arbitrarily selected at 200 and 1700 years to investigate the correlations at both early and late stages of the lifetime, and the results are presented respectively in figures 10 and 11. The presented values of the stress intensity factor are averaged with respect to the level of simulations

32  $\mathbf{x}_2$  for figures 11 and 12 (a. and b.) and the level  $\mathbf{x}_1$  for figures 11 and 12 (c. and d.) as introduced in

**1** section 3.6.

At 200 years, a sharp and direct dependence is visible between the mean value of the stress intensity factor ( $K_{1mean}$ ) and the corrosion rate (fig. **11.a**), proving that the corrosion rate has a strong influence

3 factor  $(K_{1mean})$  and the corrosion rate (fig. **11.a**), proving that the corrosion rate has a strong influence 4 on the model response. On the contrary, the mean value of the stress intensity factor has a weak

5 dependence on the contact pressure at early stages (fig. 11.b) (i.e. increasing or decreasing the contact

6 pressure seems not to have any influence on the stress intensity factor variation). This result is linked to

7 the buckling time of the liner, which is uncertain because it depends on the corrosion rate, but at early

8 stages it is very likely that the buckling of the liner has not yet occurred. Therefore, the contact pressure

9 has a very weak influence on the output because in most of the simulations, the overpack is only

- subjected to an external fluid pressure and all the variability may be associated with the uncertainties of
- 11 the corrosion rate.
- 12 The pattern exhibited by the scatterplot of the position angle  $\theta_p$  (fig. 11.c) is as well distinctive of

13 variables having no correlation at all. For the orientation angle  $\theta_0$  a really sharp dependence is visible

again (fig. 11.d), the correlation is not linear, it is more likely to be periodic, considering the nature of

15 the involved random variable. It proves that the orientation angle has a strong influence on the mean

16 stress intensity factor at the early stages of the lifetime. This result is linked as well to the buckling of

17 the liner. Before buckling, the mechanical loading is only a uniform external pressure. Therefore, the

18 stress field does not depend on the circumferential position, and neither does the mean stress intensity

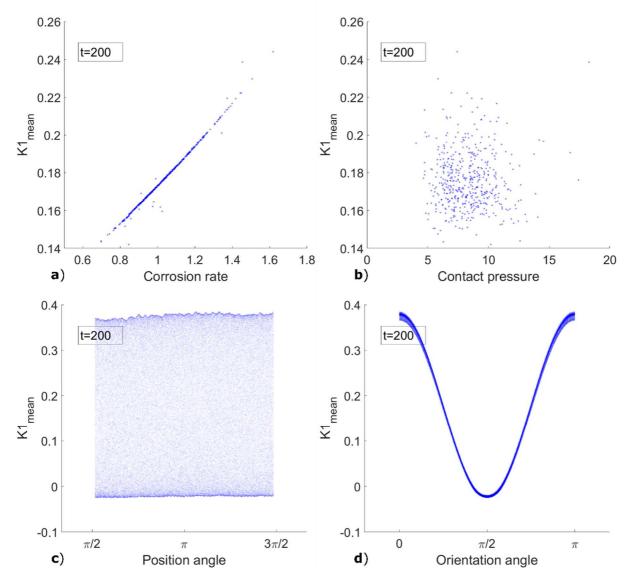


Figure **11.** Scatterplot of the mean stress intensity factor  $[MPa\sqrt{m}]$  at 200 years with respect to: **a.** the corrosion rate; **b.** the contact pressure [MPa]; **c.** the position angle; **d.** the orientation angle.

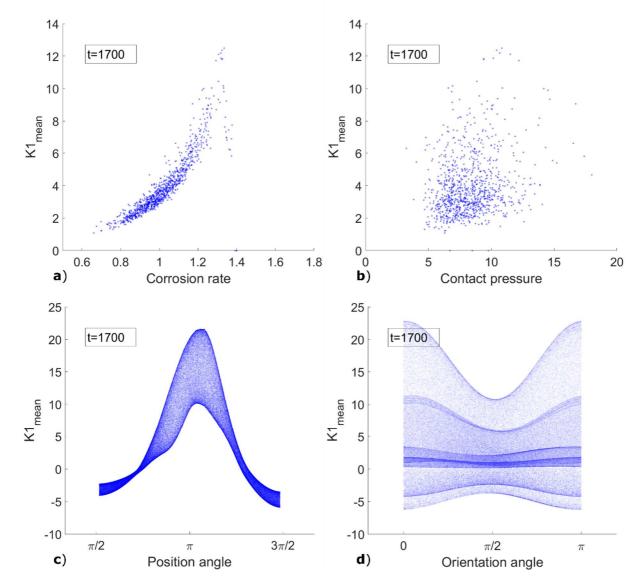
At 1700 years, the conclusions about the corrosion rate and the contact pressure are mainly the same as before: the corrosion rate has a stonger influence on the mean stress intensity factor than the contact pressure even though it is observed that the contact pressure has a slight influence (fig.12.a, b). This result proves that even after the liner buckling, the corrosion rate is more critical than the contact pressure.

10 Regarding the position and orientation angles of the crack, the conclusions drawn at 200 years are no 11 longer valid. A strong relationship can be observed between the position angle and the mean stress 12 intensity factor; and the relationship with respect to the orientation angle is weaker and seems multi-13 modal (fig. 12.c, d). The result can be explained mainly by the non-uniform mechanical loading. Once 14 the buckling of the liner occured, the stress field in the material depends strongly on the circumferential 15 position. Therefore, the position of the crack has a significant influence on the model response. In the 16 mean time, the orientation of the crak still implies a variation of the average stress intensity factor, but 17 the interval in wich these variations are contained is driven by the crack position. After a while, the 18 variations due to the position angle become significantly bigger than the variations due to the orientation 1 angle and the influence of the latter paremeter decreases. It can also be noted that the asymetry in the

2 pattern of the scatterplot of the position angle is due to the non uniform thickness of the overpack

3

induced by the corrosion process.



5 Figure **12**. Scatterplot of the mean stress intensity factor  $[MPa\sqrt{m}]$  at 1700 years with respect to: **a**. the corrosion rate; **b**. the contact pressure [MPa]; **c**. the position angle; d. the orientation angle.

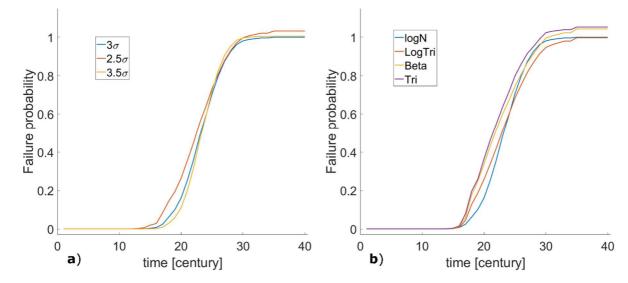
7

4

## 8 4.3. Impact of the choice of the distributions9

10 The impact of the choice of the distributions is studied herein by changing the distributions of both of 11 random variables obtained using expert judgement: the corrosion rate and the contact pressure. The 12 failure probability curves resulting from different choices of standard deviation of the corrosion rate and 13 the contact pressure are presented in figure 13.a. As detailed in section 3.7 two alternative distributions 14 considering that the amplitude of the credible interval is five and seven times the standard deviation are 15 studied. They are respectively referred to as the  $2.5\sigma$  and  $3.5\sigma$  distributions and are compared to the 16 reference  $3\sigma$  distribution. The standard deviation of the 2.5 $\sigma$  distribution is greater than the reference 17 one, therefore the resulting failure probability is also greater. For the same reason, the  $3.5\sigma$  distribution 18 induces a smaller failure probability. The re-weighting method employed is less precise when the

standard deviation of the simulated distribution is greater than the reference distribution. That is the 2 reason why the failure probability curve for the  $2.5\sigma$  distribution is less smooth than the reference one.



4 Figure 13. Failure probability considering alternative distributions for the corrosion rate and the contact pressure: a. the 2.5, 5 6 3 and 3.5 olognormal distributions; b. the reference lognormal distribution, a log-triangular, a beta and a triangular distribution.

7 The failure probability resulting from the corrosion rate and the contact pressure following triangular, 8 log-triangular and beta distributions are presented in figure 13.b. The distribution that shifts the curve 9 the most is the triangular distribution, and all of the tested distributions lead to an increase of the early 10 failure probability. However, the effect is sufficiently limited to not contradict the conclusions about the 11 reliability of the system at 500 years. All the simulated failure probability curves exhibit a long-term 12 stabilised value close to 1, the observed scatter is due to the re-weighting method and the limited amount 13 of available data.

14

3

#### 5. Conclusions 15

#### 16

17 A methodology is presented for the evaluation of the failure probability of HLW overpacks. A stochastic 18 and time-dependent structural model is developed. The finite element model takes into account the 19 corrosion process, the mechanical loading, the presence of a surface crack and the uncertainties in the 20 fracture mechanics properties of the material. The uncertainties related to these phenomena are studied 21 using five random variables for which the distributions are elicitated from expert's judgement.

22 A reliability analysis is performed with a two-level Monte Carlo method in order to reduce the numerical

23 efforts. From these numerical investigations it was found that the failure probability of the system at

24 500 years is small enough to ensure the system safety target. The first observed failures are at 1100 years

- 25 inducing a very low failure probability.
- 26 Sensitivity analyses are conducted to check the first order impact of the input parameters on the
- 27 uncertainties of the structure response. It is observed that the corrosion rate is the most critical parameter.
- 28 Therefore, this phenomenon needs to be further studied in case where more accurate estimation of the
- 29 failure probability becomes necessary.
- 30 Many distributions can result from expert's opinions elicitation, several different distributions are tested
- 31 and the influence of them on the failure probability is observed. This analysis is conducted using a re-

weighting method to avoid further evaluations of the model. The results do not contradict theconclusions about the reliability of the system at 500 years.

Further studies are still ongoing, among them, different life scenarios of the overpack are consideredplausible and should be tested, depending for example on different resaturation/desaturation kinetics of

5 the disposal cell. Moreover, different failure scenarios corresponding to the ductile fracture of the

6 overpack will be tested. This failure mode, which will however result in a lower failure probability than

7 brittle fracture, is indeed more likely to occur due to the very low sensitivity to SCC of non-alloy steels

8 in repository condition.

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10

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