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Matthieu Reymond, Jerome Sercombe, Laurent Gallais, Thomas Doualle, Yves Pontillon. Thermomechanical simulations of laser heating experiments on UO_2. Journal of Nuclear Materials, 2021, 557, pp.153220. 10.1016/j.jnucmat.2021.153220 . hal-03466237

HAL Id: hal-03466237

https://hal.science/hal-03466237

Submitted on 25 Feb 2022

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Thermo-mechanical simulations of laser heating experiments on UO₂

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Abstract

In this paper, a thermo-mechanical model suitable for the interpretation of laser heating experiments simulating the power increasing phase of a RIA transient is detailed. The thermal model includes a description of the heat deposition on the sample surface and of the heat transfer within the thickness and radius of the UO₂ sample. The mechanical modeling describes cracking and crushing of the material depending on the loading mode (tension or compression dominated) and on temperature. The brittle - ductile transition in compression is in particular well reproduced. The parameters fitting is based on available uniaxial compression tests performed at high strain rates and on bending tests performed at moderate strain rates. The application of the mechanical model to the annular laser heating experiments led to a good estimation of the crushing and cracking location (near the sample periphery), of the crack orientation (circumferential) and of the beginning of cracking.

Keywords: Nuclear fuel modeling, Damage mechanics, Laser heating, Reactivity Iniated Accident

1. Introduction

- Modeling of nuclear fuel in off normal conditions such as Reactivity Ini-
- 3 tiated Accident (RIA) or Loss Of Coolant Accident (LOCA) is necessary
- 4 to predict the response of the fuel under such extremes solicitations. Sim-
- 5 ulations of RIA transients are worldwide performed by dedicated fuel per-
- 6 formance codes such as ALCYONE at CEA [1, 2, 3] and others including
- ⁷ SCANAIR [4, 5], FRAPTRAN [6], RANNS [7], FALCON [8]. Mostly ded-
- 8 icated to the description of irradiated fuel rod thermomechanics, RIA fuel

performance code validation relies on experimental data coming from integral experiments (i.e., on fuel rodlets) performed in research reactors such as CABRI [9] in France or the NSRR in Japan [10]. In spite of the generally good reproduction of experimental trends by the fuel performance codes, some mechanisms remain difficult to tackle and require more research activity. As pointed out in the recently published synthesis report on RIA [11], this is in particular the case of fission gas release during RIA which is mostly dependent on burnup and pulse characteristics (maximum power, width). The mechanism by which fission gas release proceeds during fast power transients is thought to be grain boundary cracking related to fission gas bubbles overpressurization.

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While providing crucial information, integral experiments on fuel rods cannot be used to perform parametric studies on one phenomenon since they are complex and costly. At CEA, to reproduce RIA transient thermal conditions, a laser-heating technique usable in a hot-cell environment is under development [12]. One of the main outcome expected from these experiments is to obtain valuable data regarding the mechanical behavior of nuclear fuel under the specific loading conditions taking place during a RIA (i.e., high temperatures and strain rates) with a focus on the mechanics behind the fragmentation and grain boundary cracking of the fuel and fission gas release, especially in the restructured region of the pellet (High Burnup Structure). The impact of various parameters such as irradiation damage or fission products on the phenomenology of fission gas release could be investigated in the future. Indeed, it is not clear from past integral experiments if the kinetics of fission gas release during a RIA transient is one of the driving parameters that can lead to rod failure. As such, it is important to propose complementary experiments on irradiated fuel that can be multiplied for parametric studies.

In view of the objective of the laser heating experiments, numerical simulation must be developed at the same time. It should furthermore not be limited to the thermal behavior of the fuel but should also include a precise description of the mechanical behavior of the fuel since it will be of primary importance for the identification of fission gas release mechanisms. In a first step towards this objective, this paper presents the development and validation of a mechanical behavior law for non irradiated UO₂. Laser-heating experiments are used to investigate the relation between fuel fragmentation (grain boundary and macroscopic cracking) and thermomechanical loading representative of RIA conditions. A sophisticated material model is thus

proposed to describe micro- and macro-cracking of UO₂ fuel both in tension and compression within a large range of temperatures. After a presentation of the laser heating experiments and of the material model, the simulation results are discussed and compared to on-line measurements.

2. Laser heating experiments

A Reactivity Iniated Accident thermal transient is characterized by the fast rise of power and of temperature that occurs in the fuel. In irradiated fuels, the increase of power and temperature can be strongly localized at the periphery of the pellet. Typical temperature increase obtained in research reactors are around 12273-1773 K (1000-1500°C) at the fuel pellet centerline and up to 2273 K (2000°C) at the pellet periphery, as shown in Figure 1. The temperature increase is quasi-adiabatic and happens in a few tens of milliseconds.

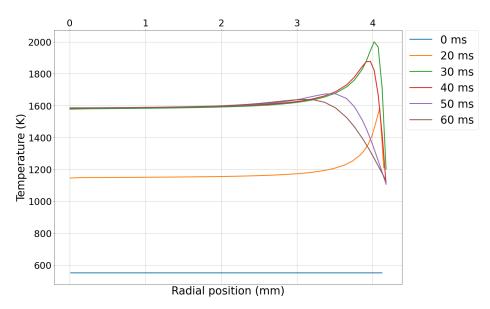


Figure 1: Radial temperature distribution calculated with ALCYONE fuel perfomance code during the NSRR RH-2 test (adapted from [1])

The capacity of laser-heating to reproduce RIA thermal transient at laboratory scale is based on the use of a high power Ytterbium fiber laser. Encouraging results have been reported in [12] with temperature as high as 2273K (2000°C) being reached at the periphery of the sample in the few tens of milliseconds. UO_2 being a strongly absorbing material at the wavelength used in the setup (1080 nm) [13, 14], laser power absorption is localized at the surface of the samples. Accordingly, with a time scale of a few tens of milliseconds, the sample thickness must be submillimetric if an homogenous heating of the sample is expected. The samples are therefore prepared from fuel pellets in the form of disks less than 1 mm thick.

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The experiments presented in this paper are characterized by an annular heating of the samples as shown in Figure 2.

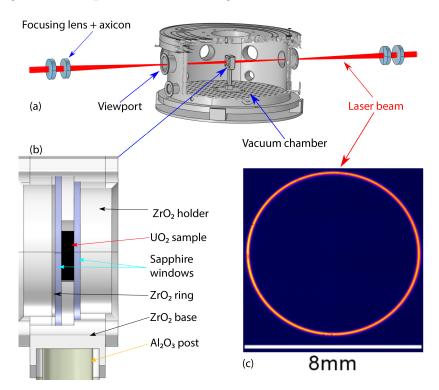


Figure 2: Sketch of the experimental chamber (a) and sample holder (b) used in the laser heating experiments. (c): Intensity profile of the laser beam obtained in the sample plane

The sample, which is obtained from UO_2 fuel pellets fabricated at CEA using a wet process followed by isostatic compaction at 450 MPa and sintering at 1650°C during 2 h under flowing H_2 , is placed vertically in a sample holder made of zirconia with two optical sapphires that ensure optical access to the sample and retain any possible fragments in case of loss of integrity. The disk, sample holder and sapphires are placed in a dedicated experimental chamber

operated in vacuum (10^{-2} mbar) which allows monitoring and optical access through multiple viewports. The sample is first maintained at a starting temperature by a gaussian beam. It is then heated at its periphery by one or two annular gaussian beams of pre-selected dimensions (radius, waist). While not being strictly representative of a RIA, these experiments lead nevertheless to high strain rates and temperature increase rates that are of the order of magnitude of those encountered during fast power transients.

Two sets of experiments are presented in this paper. The first one is a series of 20 ms long laser heating experiments conducted on a 970 μ m thick sample starting from a homogeneous temperature state of around 800°C. Different powers are used, ranging from 150 to 1200 W with both faces of the sample being heated by two identical annular beams. The beam radius and waist are 3.5 and 0.5 mm, respectively. Real time temperature measurement by infrared imaging with a high speed thermal camera has been recorded during this series of experiments as showed on Figure 3 and will therefore be used to validate the thermal part of the simulations.

The second series consists of five heating experiments of duration ranging from 10 to 100 ms. The last one was performed with 300 W of laser power while the ones before used 150 W. These experiments were conducted on the same sample of thickness 930 μ m and started from room temperature. Beam radius and waist are 3.9 and 0.1 mm, respectively. During these experiments, only one face of the sample is heated. Failure of the sample periphery happened brutally after around 40 ms of heating, as determined from the pictures taken with a high speed visible camera. The post experiment state of the sample is showed Figure 3.

The last two experiments of the second series (i.e., 100 ms - 150 W and 100 ms - 300 W) will be used to validate the fuel mechanical behavior law presented in the next part.

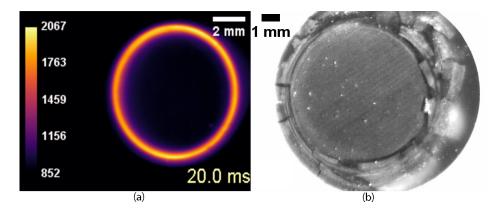


Figure 3: (a) False color image and corresponding temperature scale (in $^{\circ}$ C) obtained with the high speed infrared camera during the first series of experiments. Laser power was 900W and duration of the heating was 20ms. (b) Sample with a clear circumferential cracking of the periphery after the 100 ms - 300 W laser heating experiment of the second series of experiments

3. Mechanical behavior law for UO₂ submitted to laser heating

As most ceramics, uranium dioxide presents a brittle behavior at low and moderate temperature both in tension and compression. However, the fracture stress is dissymmetric: at room temperature, it can reach 1000 MPa in compression while it is around 100 MPa in tension. At higher temperatures, the behavior of UO₂ becomes ductile. The brittle-ductile transition is not only dependent on temperature but also on the strain rate. In general, the higher the strain rate, the higher the temperature of transition from fragile to ductile behavior will be both for the compression domain [15, 16] and the tensile domain [17].

Salvo et al. obtained a brittle-ductile transition between 1100° C and 1350° C at a strain rate of 10^{-1} /s during uniaxial compression tests [15, 16]. A ductile behavior with pronounced grain boundary cracking was observed at temperatures $\geq 1550^{\circ}$ C leading compressive stresses close to 100 MPa. A brittle behavior with cracks parallel to the stress direction occurred at moderate temperature (1100°C).

Similar observations have been reported for the tensile domain by Evans et al. [18]. They identified a temperature of transition of around 1473 K (1200°C) from three points bending tests. Similarly, Canon et al. [17] found a temperature of transition of ~ 1473 K (1200°C) from four points bending tests leading to an ultimate tensile stress of ~ 120 MPa and to a strain rate of 9.2/h ($2.10^{-3}/s$). With strain rates usually exceeding 1/s during RIA transients, a brittle behavior is expected in tension even at high temperatures where the behavior would normally be ductile at lower strain rates.

To model the dissymetric mechanical behavior of UO_2 in tension and compression, the damage μ -model proposed by Mazars et al. [19], classically used for concrete, is considered. It is an improved version of the first damage model developed by Mazars [20]. Damage is described by a scalar variable d (evolution between 0 and 1) that is used to model the loss of stiffness of the material. Two distinct damage modes are considered for traction and compression, i.e. cracking and crushing, respectively. As such, both compressive and tensile loadings can lead to the development of damage and unilateral effects such as cracks opening and closure if the loading is reversed are considered in the model. Moreover, damage development is related to the first and second invariants of the strain tensor, meaning that the crack orientations are not fixed a priori in this model. This approach is more general than those used in many mechanical models available in fuel

performance codes [2], [21, 22] where the crack directions are prescribed by the cylindrical coordinate system.

In the μ -model the stress-strain relation is defined by:

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$$\sigma = (1 - d) \mathbf{C} : \epsilon \tag{1}$$

with σ the standard Cauchy stress tensor, \mathbf{C} the elastic tensor and ϵ the elastic strain tensor.

To model damage, the equivalent strain concept is used. Cracking and crushing are respectively associated to the following equivalent strains:

$$\epsilon_t = \frac{I_\epsilon}{2(1 - 2\mu)} + \frac{\sqrt{J_\epsilon}}{2(1 + \nu)} \tag{2}$$

 $\epsilon_c = \frac{I_\epsilon}{5(1 - 2\mu)} + \frac{6\sqrt{J_\epsilon}}{5(1 + \nu)} \tag{3}$

with I_{ϵ} and J_{ϵ} as the first and second invariants of the elastic strain tensor and ν Poisson's ratio.

Two loading surfaces are then defined:

$$f_t = \epsilon_t - Y_t \quad \text{and} \quad f_c = \epsilon_c - Y_c$$
 (4)

with Y_t and Y_c two thermodynamic variables related to the maximum equivalent strains reached during the loading sequence:

$$Y_t = \operatorname{Sup}[\epsilon_{0t}, \max(\epsilon_t)] \quad \text{and} \quad Y_c = \operatorname{Sup}[\epsilon_{0c}, \max(\epsilon_c)]$$
 (5)

Before damage onset, Y_t and Y_c are equal to the initial thresholds ϵ_{0t} and ϵ_{0c} . The effective damage d can then be estimated from the thermodynamic variables Y_t and Y_c as follows:

$$d = -\frac{(1-A)Y_0}{Y} - A.\exp(B(Y - Y_0))$$
(6)

with Y_0 the initial value of Y defined by:

$$Y = rY_t + (1 - r)Y_c \tag{7}$$

and A, B two parameters that are described by the following functions:

$$A = A_t(2r^2(1-2k) - r(1-4k)) + A_c(2r^2 - r + 1)$$
(8)

$$B = r^{(r^2 - 2r + 2)} B_t + (1 - r^{(r^2 - 2r + 2)} B_c)$$
(9)

with r the triaxial factor indicating the nature of the stress. r=0 corresponds to a uniaxial compression state and r=1 to a uniaxial tensile state. When $r=1, A=A_t$ and $B=B_t$ and when $r=0, A=A_c$ and $B=B_c$. k is a parameter that defines the asymptotic stress during pure shear loading, see reference [19] for more details.

The four parameters A_t , B_t , A_c and B_c can be easily identified from strainstress curves obtained from uniaxial compression and tensile or bending tests. Two strain thresholds activating the damage evolution need also to be defined for tension and compression, i.e., ϵ_{t0} and ϵ_{c0} , respectively.

The compression behavior of the model has been fitted on the strain-stress curves of M. Salvo et al. [15, 16] obtained during uniaxial compression tests at high strain rates $(10^{-1}/s)$ and high temperatures (1373-2023 K) representative of RIA loading conditions and which clearly led to a brittle-ductile transition. The resulting model parameters are summarized in Table 1. The extrapolation of the evolution of the parameters towards room temperature is based on other existing uniaxial compressive tests [23, 24]. A decrease of the compressive yield stress at temperatures higher than 1973 K (1700°C) has been assumed with a target of 60 MPa around the melting temperature of UO_2 ($\sim 3000\text{ K}$). The tensile behavior is assumed to remain brittle whatever the temperature reached and is characterized by a constant ultimate tensile stress of around 80 MPa. Due to the lack of experimental data, the asymptotic shear stress has been assumed null, leading to k=1.

Parameter	Expression
$\overline{A_t}$	3.2
B_t	2850
ϵ_{t0}	0.00001
$\overline{A_c}$	T< 1500 K:
	$A_c = 8.9719 \times 10^{-7} \times T^2 - 3.11509 \times 10^{-3} \times T + 2.8357$
	$T \ge 1500 \text{ K}$:
	$A_c = 9.47303 \times 10^6 \times T^{-2.6154}$
B_c	T < 1500 K:
	$B_c = -0.828T + 173.59$
	$T \ge 1500 \text{ K}$:
	$B_c = 50$
ϵ_{c0}	T < 1500 K:
	$\epsilon_{c0} = -0.00001$
	$1500 \text{ K} \le \text{T} \le 2500 \text{ K}$:
	$\epsilon_{c0} = -4 \times 10^{-10} \times T^2 + 2 \times 10^{-6} \times T - 0.0032$
	T > 2500K:
	$\epsilon_{c0} = -0.00058$
k	1

Table 1: Parameters of the μ -model fitted to UO₂

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The model is solved numerically using the code generator Mfront [25]. A very interesting feature is the possible explicit integration of the stresses with the μ -model. A large set of loading configurations (uniaxial, cyclic, biaxial, ...) and of temperatures, particularly around the temperature of transition, has been first tested with the mtest software (ensuring a calculation at a single material point) included in the MFront code generator. Figure 4 presents the stress-strain curves obtained for uniaxial tension and compression at constant temperatures varying between room temperature and 2773 K. As can be seen, the fragile-ductile temperature transition is set at 1500 K (1227°C). At temperatures below 1500 K, the compressive strain-strain curve is characterized by a decrease of the stress upon reaching a maximum value. The residual stress in each case is zero but it is reached at an increasing level of strain with the temperature increase. In Salvo's et al. test performed at 1373 K (1100°C) [16], the maximum stress was close to 300 MPa and softening occurred till 15% of axial strain. At room temperature, the maximum stress in uniaxial compression is around -1000 MPa, consistent with the experiments in the open literature [26]. At temperatures equal or above 1500 K, the behavior is characterized by a stress peak reached between 1 and 2% of axial strain followed by a plateau at a stress level slghtly lower than the peak. This behavior is typical of UO_2 tested at high temperature and has been attributed to structural [24] or material [27] origin. In some of Salvo's tests at high temperatures, the axial strain reached 20% without any sign of softening. The tensile behavior is represented by a single stress-strain curve since no temperature dependency has been considered in this work. The maximum stress reaches 80 MPa and is followed by a fast decrease of the stress leading a zero-stress value at an axial strain of 0.1%. For the behavior of the μ -model during biaxial loading conditions (compression-compression, compression-tension or tension-tension) and the unilateral aspects (stiffness recovery when compression loading follows tensile loading, i.e., cracks closure), the interested reader may refer to the original paper of Mazars et al. [19].

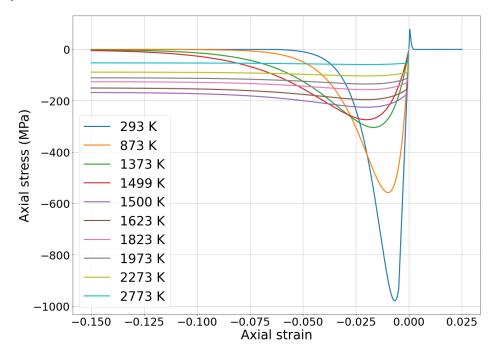


Figure 4: Stress-strain curves evolution with temperature as obtained from the μ -model. The traction behavior is independent of temperature.

One important aspect of the proposed mechanical model for UO_2 is the consequent development of damage in compression even if no softening is

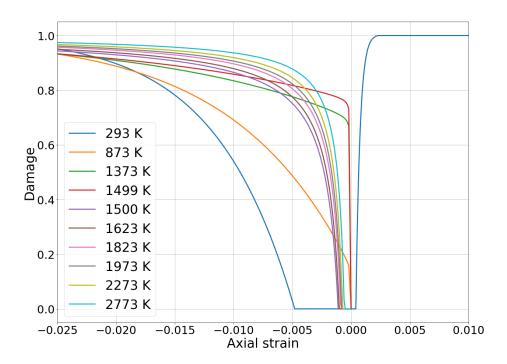


Figure 5: Damage-strain curves evolution with temperature as obtained from the μ -model. The traction behavior is independent of temperature.

obtained in the stress-strain curves. Figure 5 presents the damage evolution during the uniaxial compression and traction simulations of Figure 4. As can be seen, even at temperatures above 1500 K where the behavior is ductile, significant damage occurs with values of 0.8 or higher being reached even at a relatively low strain level. This choice is related to the pronounced grain boundary cracking that was observed in Salvo's tested samples at high temperature [16].

4. Thermo-mechanical simulations of the laser heating tests

4.1. Mesh, boundary conditions and material properties

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The simulations presented in the sequel have been performed with the Finite Elements code Cast3M [28] developed by the CEA and used in the fuel performance code ALCYONE [29]. A 2D axisymmetric representation of a cross-section of the fuel disk is meshed. For the experiments with both disk faces heated, half of the sample thickness is considered while the whole thickness is considered for the experiments where only one face is heated.

Given the short duration of the laser heating experiments, thermal losses by radiative exchange, convection or possible losses related to the sample holder are not considered. The mesh of the fuel disk cross-section (half or quarter depending on the heating conditions) is illustrated in Figure 6 together with the thermal and mechanical boundary conditions. The mesh is especially refined below the laser heated surface (heat flux q_0). The mesh density is then gradually increased with the distance from this zone. Mechanical displacements in the vertical direction at the bottom of the mesh are blocked.

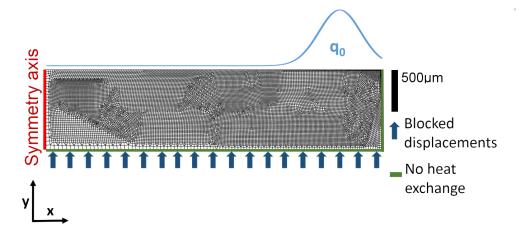


Figure 6: Mesh of the fuel pellet disk and thermo-mechanical boundary conditions considered

The fuel thermal and mechanical properties used in the simulations are relative to non irradiated UO_2 and extracted from the open literature [30, 31, 32]. The temperature dependent expressions are summarized in Table 2.

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Property	Expression
Thermal conductivity $(W.m^{-1}.K^{-1})$ [30]	$\lambda(T) = \frac{10^2}{7.5408 + 17.692t + 3.6142t^2 + \frac{6400}{t^{5/2}} \exp(\frac{-16.35}{t})}$ with $t = T/1000$
Heat Capacity $(J.mol^{-1}.K^{-1})$ [30]	298.15 K $\leq T \leq$ 3120 K: $C_p(T) = 52.1743 + 87.951t - 84.2411t^2 + 31.542t^3 - 2.6334t^4 -0.71391t^{-2}$ with $t = T/100$
Density (kg/m^3) [30]	$\rho(T) = \rho_{273} \left(\frac{L(T)}{L_{273}}\right)^3$ with $\rho_{273} = 10963$ kg/m ³ , the density at 273 K
Thermal expansion coefficient (K ⁻¹) [30] from [33]	$\alpha = \frac{1}{L} \left(\frac{dL}{dT} \right)$ $273 \text{ K} \le T \le 923 \text{ K: } L(T) = L_{273} (9.9734 \times 10^{-1} + 9.802 \times 10^{-6}T - 2.705 \times 10^{-10}T^2 + 4.391 \times 10^{-13}T^3)$ $923 \text{ K} < T \le 3120 \text{ K: } L(T) = L_{273} (9.9672 \times 10^{-1} + 1.179 \times 10^{-5}T - 2.429 \times 10^{-9}T^2 + 1.219 \times 10^{-12}T^3 \text{ K}^{-1})$
Young Modulus (GPa) [34] Poisson Ratio	$E(T) = (1 - 2.5p)(2.2710^2 - 1.54 \times 10^{-2}T - 9.60 \times 10^{-9}T^2)$ 0.3

with T the temperature in K and p the porosity

Table 2: UO_2 thermophysical and elastic properties used in the simulations

4 4.2. Thermal loading and simulation

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In the considered axisymmetric configuration, the laser heat flux q_0 (in W/m^2) represented in Figure 6 can be expressed as follows:

$$q_0(r,t) = (1 - X).f(t).I(r, \omega_{an}, R_{an}, P)$$
(10)

It is thus defined by the following terms:

- a time function f(t) indicating whether the laser power is on or off(1 or 0),
 - the intensity distribution I, in (W/m^2) , which is a function of r the radial position and of the characteristics of the annular beam (i.e., beam waist ω_{an} , radius R_{an} and power P),
 - a coefficient accounting for the losses along the optical path (1 X), X being the sum of the losses along the optical path, including the reflectivity of the sample and of the sapphire window.

The intensity distribution of the annular gaussian beam I (in W/m²) is given by the following expression:

$$I(r, R_{an}, \omega_{an}) = I_{max} \exp\left[-\frac{2(r - R_{an})^2}{\omega_{an}^2}\right]$$
(11)

with I_{max} (W/m²) the peak intensity of the Gaussian distribution:

$$I_{max} = \frac{P}{\pi \left(\frac{\omega_{an}^2}{2} \exp\left[-\frac{2R_{an}^2}{\omega_{an}^2}\right]\right) + R_{an} \frac{\omega_{an}\sqrt{\pi}}{\sqrt{2}} \operatorname{erfc}\left(-\frac{\sqrt{2}.R_{an}}{\omega_{an}}\right)}$$
(12)

where erfc is the complementary error function and P the laser power given in W.

To check the thermal modeling of the laser heating test, the first test series has been simulated. The initial uniform temperature of the disk equals 1123 K (850°C). The heat flux q_0 resulting from the laser beams on each side of the disk (P = 620 W, $R_{an} = 3.5$ mm and $w_{an} = 0.5$ mm) is then prescribed during 20 ms. The calculated radial temperature profiles at the disk surface are then plotted every 5 ms, as shown in Figure 7. A good agreement between the

calculated radial profiles of temperature and the measurements during the experiments by infrared imaging is obtained. In particular, the calculated peak temperature is close to the measured one. Moreover, these results are in agreement with the temperature profiles calculated with the COMSOL solver used by Vidal et al. to design these laser heating tests [12].

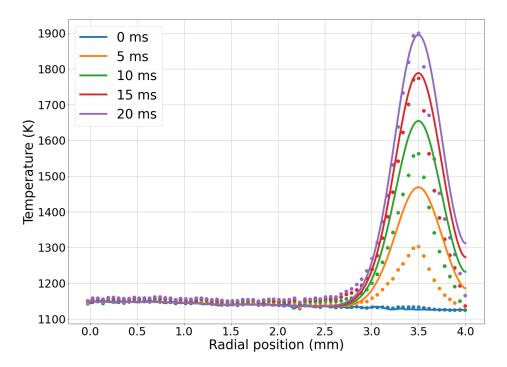


Figure 7: Simulated (continuous lines) and measured (dots) surface temperatures during a 20 ms duration and 620 W laser heating experiment

Simulations accounting for radiative heat exchanges between the sample and the sapphire windows have been done using values of the total hemispherical emissivity available in the literature for both UO_2 and sapphire [32, 35]. Radiative heat exchanges had a negligible impact on the calculated temperatures. These results confirmed that they can be neglected in the simulations.

4.3. Thermo-elastic simulation

The second test series of laser heating experiments is now considered. Thermo-elastic simulations are first performed. The initial temperature of the sample is in this case the room temperature. The calculated peak surface

temperature reaches 973 K (700°C) during the 100 ms - 150 W test and 2273 K (2000°C) during the 100 ms - 300 W test, as shown in Figure 8. Heating takes place only on one of the disk face. No temperature measurement is available.

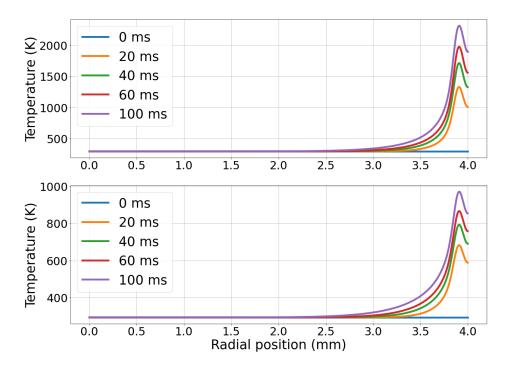


Figure 8: Calculated evolution of the surface temperature during the $100~\mathrm{ms}$ - $300~\mathrm{W}$ (top) and $100~\mathrm{ms}$ - $150~\mathrm{W}$ (bottom) experiments

The calculated strain rates show that these experiments are relevant for the study of RIA transient mechanisms since the strain rates in the heated zone reach more than 3 /s during the 100 ms - 300 W experiment. The calculated radial profiles of the radial, hoop and axial stresses on the heated surface (peak laser power) during this test are plotted in Figure 9.

Significant compressive peaks are obtained for the two first stress components near the pellet periphery. The stress values are however unrealistic. The maximum hoop stress reaches -6000 MPa. As detailed in part 3, the compressive strength of UO₂ at room temperature does not exceed 1000 MPa. Looking now at the radial stress distribution in the disk presented in Figure 10, one may note that a strong radial compression state holds in the heated region of the sample (circled in Figure 10) and a concomitant radial tensile

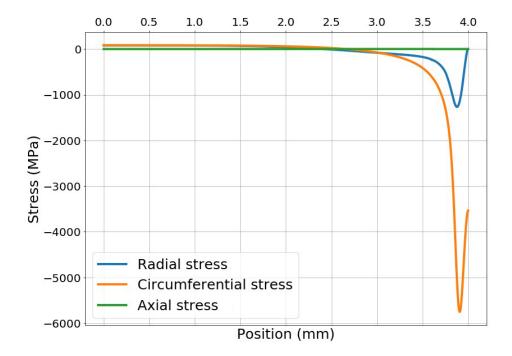


Figure 9: Thermo-elastic radial, hoop and axial stresses radial profiles at the heated surface of the disk after $100~\mathrm{ms}$ of annular laser heating at $300~\mathrm{W}$.

state is observed in the area beneath it.

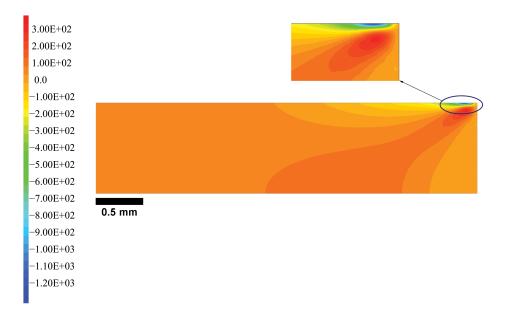


Figure 10: Calculated radial stress distribution (in MPa) within the fuel disk after 100 ms of annular laser heating at 300 W on the top surface

The magnitude of these tensile radial stresses depends on the first order of the compressive stresses in the heated region of the disk. Radial stresses being the driving force behind circumferential cracking, it is of utmost importance to precisely model the behavior of UO_2 in compression and during fast temperature rise if one wants to assess the fragmentation of the fuel during the laser heating experiments. With the high thermo-elastic compressive stresses reached in the heated zone, the calculated tensile stresses in the region (almost 250 MPa) beneath exceed the UO_2 tensile strength. Circumferential cracking of the sample during the heating phase is thus expected.

4.4. Thermo-mechanical analysis with the μ -model

The μ -model was then used for the simulation of the two 100 ms experiments of the second test series. During the simulation of the 150 W experiment, damage develops only in the heated area. The damage level reached is moderate with a maximum close to 0.5. As expected, this damage is the consequence of the high compressive stresses reached in the heated area, as shown by the isovalues of Y_c given in Figure 11. Values of Y_t during this experiment remain below 10^{-4} . Crushing is thus the damage mode that take place in the laser heated area.

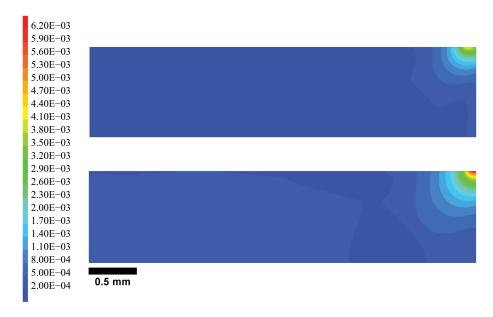


Figure 11: Calculated Y_c values at 40 (top) and 100 ms (bottom) during the simulation of the 150 W - 100 ms experiment

During the simulation of the 300 W experiment, two distinct damage zones are observed before the failure of the sample (at 40 and 45 ms), as shown in Figure 12.

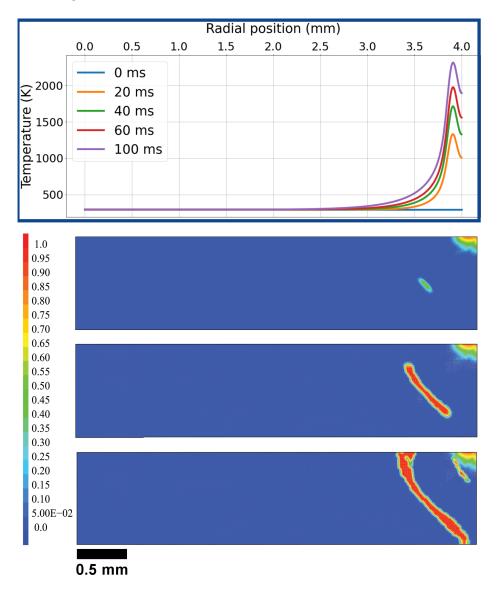


Figure 12: Calculated damage values at 40, 43 and 46 ms during the simulation of the 100 ms - 300 W experiment. Top graph: calculated radial temperature profiles at various times

As for the simulation of the 150 W test, the area directly heated by the laser is damaged by crushing (local compression). The area beneath the heated zone is on the contrary damaged by the development of tensile stresses that exceed the material strength (cracking). This is confirmed by the isovalues of Y_t , plotted in Figure 13 at different times during the experiment.

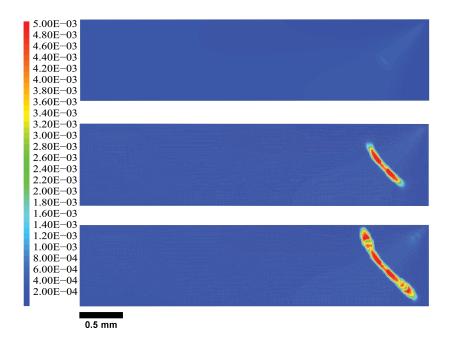


Figure 13: Calculated Y_t values at 40, 43 and 45 ms during the 300 W - 100 ms experiment. The maximum reached at 45ms is 8E-03.

At 39-40 ms, Y_t starts to increase beneath the heated area where the Y_c variable is dominant (slight increase visible on Figure 13). This simulation result is consistent with the radial stress distribution calculated in the thermo-elastic simulation of the test, see Figure 10. In consequence of the high tensile radial stresses reached beneath the heated area, a circonferential fracture develops as can be seen in Figure 12. A good agreement between the calculated (46 ms) and recorded circumferential cracking time (~ 40 ms) of the disk is obtained, showing the good capability of the model to correctly assess the compressive stresses in the heated zone and the tensile counterpart. The post-mortem SEM examination of the sample shown in Figure 14 confirmed the development of a circumferential crack near the pellet periph-

ery and the micro-damage of the material in the heated area that had led to some visible grain boundary cracking.

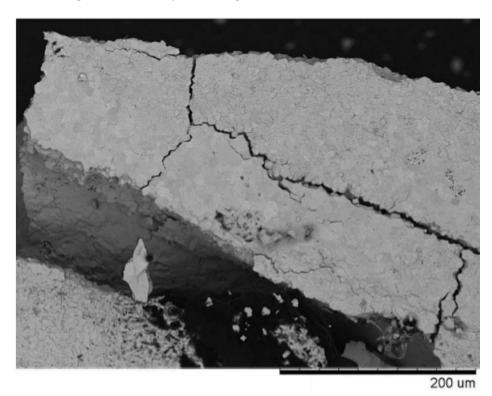


Figure 14: SEM examination of the UO_2 disk periphery after the 300 W - 100 ms experiment

5. Discussion

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Thermo-mechanical simulations of the behavior of UO₂ samples submitted to high-power laser heating have been presented in this paper. In a first part, the relevance of the thermal and mechanical models are discussed. In a second part, the applicability of the proposed laser heating technique to irradiated nuclear fuel and fission gas release studies during RIA simulated thermal transients is discussed.

5.1. Relevance of the thermal and mechanical models

The radial temperature profile and peak temperature are well reproduced by the thermal model implemented in this work even if the thermal boundaries conditions do not account for heat exchange with the surrounding materials. The shape of the radial temperature profile is however not perfectly reproduced since the calculated profile is slightly wider than the experimental one.

This fact may come from a discrepancy between the theoretical intensity distribution of an annular gaussian beam (Eq.11) and the one obtained in our setup. It is therefore also possible to use the experimental intensity profile of the laser (see insert on Figure 2) instead of the theoritical one. Such profile measurements were not available for the presentend experiments.

A discrepancy between the measured and calculated temperatures during the first few milliseconds of the experiment can also be observed. It is supposed to be induced by the temporal incertitude of the measurements. The laser pulse and thermal camera were not synchronized and with an acquisition rate of 5 ms, the incertitude can then be as high as 5 ms. In future work, the laser pulse and thermal camera will be synchronized.

The proposed mechanical model is able to reproduce the mechanical behavior, damage and fracture mechanisms of uranium dioxide submitted to laser-heating experiments (crushing and cracking with stress relaxation). As shown in this paper, the craking time experimentally recorded is well reproduced with this model provided the model parameters for both tension and compression are well defined from material tests performed at high strain rates and temperatures. While the incertitude from the visible camera rate of acquisition of 800 images/s is inferior to 1.25 ms, one could argue that the definition of the instant of cracking from highly saturated images such as the ones swhowed in Figure 15 is arbitrary. As can be seen in Figure 15 from the thermal radiation recorded by the camera during the test, it is however sure that cracking occurred between 36 and 45 ms. A conservative assessment then leads to choose 40 ms as the time where the sample begins to lose its integrity.

To lift uncertainties regarding the beginning of cracking, other measuring devices less prone to saturation could be used by using an optical device filtering thermal radiation. Future works will then aim to further validate the cracking criterion with a better determination of the time and visualisation of the cracking. While it is true that the definition of the beginning of cracking from the showed image sequence will highly depend on the chosen criterion, it is interesting to point out that this instrumentation clearly showed that cracking happened during the heating phase and not during the cooling phase. Cracks and fragments indicated by blue arrows in Figure 15

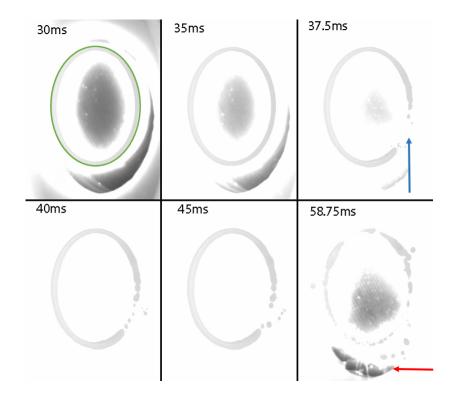


Figure 15: Evolution of the thermal radiation recorded by a high-speed visible camera during the 100 ms - 300 W experiment. The green circle roughly indicate the periphery of the sample. Sample's radius is 4mm. The blue arrow point out the beginning of the fracturation. The red arrow indicate visible fragments seen during the experiment.

are in fact clearly visible after 50 ms. In thermal loading sequences, one usually expects significant damage to take place during the cooling phase rather than during the heating phase. The occurence of damage during the heating phase could not have been detected without a proper instrumentation. It shows the importance to perform out-of-pile experiments at the lab scale with sophisticated instrumentation, such instrumentation being easier to implement than in research reactors.

5.2. Applicability to irradiated nuclear fuel

From the analysis of the stress state in the sample during the laster experiments, it is clear that the area of the sample directly heated is subjected to high compressive stresses. This compression state and its tensile counterpart can lead to damage of the microstructure of the material by local crushing/cracking potentially weakening the grain boundaries. If pre-irradiated

samples were to be tested under the same loading sequence, one could expect fission gas release related to grain boundary cracking to take place, by the same mechanism than during RIA transients. The ability to induce 401 grain boundary cracking in the periphery of the sample subjected to a biaxial 402 stress state, as confirmed by SEM examination, is particularly interesting. 403 Indeed, in high burnup fuels, the fragmentation of the pellet rim is a well-404 known phenomenon piloting fission gas release during RIA transients [9]. As such, having the capacity to reproduce RIA thermal conditions and induc-406 ing equivalent stress states in some parts of the fuel sample within a heavily 407 instrumented environment could allow precise investigations regarding this 408 topic. In this respect, it should also be possible to uncouple compression and 400 tensile induced damage (i.e., inducing compression damage without tensile 410 related damage) with this technique and thus to quantify the contribution of each phenomenon to fission gas release. This could potentially give great 412 insights on fission gas release kinetics. In fact, during the power increasing phase of a RIA, the fuel periphery is first subjected to compressive stresses. 414 Whether fission gas release takes place during this period or not is of importance to assess the potential for clad ballooning and burst if the fission gas related pressure reached inside the rod is too high. The transients showed in this paper do not reproduce a full RIA thermal sequence departing from 418 normal operating conditions where a radial parabolic temperature gradient 419 holds. Future works will aim to tend towards a more representative thermal 420 loading sequence. The use of a second Gaussian beam to reproduce both 421 the dynamic increase of temperature in the inner part of the sample and the 422 parabolic shape of the temperature profile during the cooling phase of the 423 transient is currently under study. The dynamic adjustment of the power of this secondary beam should also allow a controlled cooling in order to 425 reproduce the whole RIA thermal transient.

427 6. Conclusion

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In this paper, a thermo-mechanical model suitable for the interpretation of laser heating experiments simulating the power increasing phase of a RIA transient has been detailed. The specificity of the experiments lies in the annular heating of the UO₂ disk samples that is intended to reproduce the peaked temperature radial profile in the pellet during this type of transient. The thermal model includes a description of the heat deposition on the pellet surface and of the heat transfer within the thickness and radius of the

UO₂ disk. The mechanical modeling relies on an adaptation of the damage μ -model proposed by Mazars et al. [19] for concrete materials to uranium dioxide. It describes cracking and crushing of the material depending on the 437 loading mode (tension or compression dominated) and on temperature. The brittle - ductile transition in compression is in particular well reproduced. 439 The parameters fitting is based on available uniaxial compression tests per-440 formed at high strain rates and on bending tests performed at moderate strain rates. The application of the mechanical model to the annular laser 442 heating experiments led to a good estimation of the crushing and cracking location (near the pellet periphery), of the crack orientation (circumferential) and of the beginning of cracking. More experiments with on-line tempera-445 ture measurements and image recording are however needed to better assess the tensile cracking kinetics and the loading conditions leading to crushing. The SEM analysis performed post-mortem showed the development of grain 448 boundary cracking in the heated zone. These annular laser heating experiments could therefore be used to study fission gas release mechanisms during lab simulated RIA transients.

References

- [1] I. Guénot-Delahaie, J. Sercombe, T. Helfer, P. Goldbronn, Fédérici, T. Le Jolu, A. Parrot, C. Delafoy, C. Bernaudat, Simulation of reactivity-initiated accident transients on UO₂ -M5[®] fuel rods with AL-CYONE V1.4 fuel performance code, Nuclear Engineering and Technology 50 (2018) 268–279.
- 458 [2] J. Sercombe, I. Aubrun, C. Nonon, Power ramped cladding stresses and 459 strains in 3D simulations with burnup-dependent pellet—clad friction, 460 Nuclear Engineering and Design 242 (2012) 164–181.
- [3] J. Sercombe, T. Helfer, E. Federici, D. Leboulch, T. Le Jolu, A. Hellouin de Ménibus, C. Bernaudat, 2D simulation of hydride blister cracking during a RIA transient with the fuel code ALCYONE, EPJ Nuclear Sciences & Technologies 2 (2016) 22.
- [4] A. Moal, V. Georgenthum, O. Marchand, SCANAIR: A transient fuel
 performance code Part One: General modelling description, Nuclear
 Engineering and Design 280 (2014) 150–171.
- ⁴⁶⁸ [5] V. Georgenthum, A. Moal, O. Marchand, SCANAIR a transient fuel performance code Part two: Assessment of modelling capabilities, Nuclear Engineering and Design 280 (2014) 172–180.
- [6] P. M. M.E. Cuningham, C.E. Beyer, G. Berna, FRAPTRAN: A Computer Code for the Transient Analysis of Oxide Fuel Rods, Technical Report NURERG/CR-6739 VOL.1, U.S Nuclear Regulatory Commission, 2001.
- [7] M. Suzuki, T. Fuketa, H. Saitou, Analysis of Pellet-Clad Mechanical Interaction Process of High-Burnup PWR Fuel Rods by RANNS Code in Reactivity-Initiated Accident Conditions, Nuclear Technology 155 (2006) 282–292.
- [8] W. Lyon, R. Montgomery, N. Jahingir, S. Yagnik, Capabilities of the FALCON Steady State and Transient Fuel Perfomance Code, in: Proceedings of the 2004 International Meeting on LWR Fuel Performance.
- [9] J. Papin, B. Cazalis, J. M. Frizonnet, J. Desquines, F. Lemoine, V. Georgenthum, F. Lamare, M. Petit, Summary and Interpretation of the CABRI REP-Na Program, Nuclear Technology 157 (2007) 230–250.

- [10] T. Fuketa, T. Sugiyama, F. Nagase, Behavior of 60 to 78MWd/kgU
 PWR Fuels under Reactivity-Initiated Accident Conditions, Journal of
 Nuclear Science and Technology 43 (2006) 1080–1088.
- [11] Nuclear Fuel Behaviour Under Reactivity-initiated Accident 488 (RIA) Conditions - State-of-the-art Report, Technical Report, 489 NEA/CSNI/R(2010)1, 2010. 490
- [12] T. Vidal, L. Gallais, J. Faucheux, H. Capdevila, J. Sercombe, Y. Pontillon, Simulation of reactivity initiated accident thermal transients on nuclear fuels with laser remote heating, Journal of Nuclear Materials 530 (2020) 151944.
- [13] P. Ruello, K. Becker, K. Ullrich, L. Desgranges, C. Petot, G. Petot-Ervas, Thermal variation of the optical absorption of UO₂: determination of the small polaron self-energy, Journal of Nuclear Materials 328 (2004) 46–54.
- ⁴⁹⁹ [14] K. W. M. Bober, J. Singer, Spectral Reflectivity and Emissivity Mea-⁵⁰⁰ surements of Solid and Liquid U0₂ at 458, 514.5 and 647 nm as a Func-⁵⁰¹ tion of Polarization and Angle of Incidence, Technical Report, Kern-⁵⁰² forschungszentrum Karlsruhe - Institut für Neutronenphysik undReak-⁵⁰³ tortechnik, 1980.
- [15] M. Salvo, J. Sercombe, T. Helfer, P. Sornay, T. Désoyer, Experimental characterization and modeling of UO₂grain boundary cracking at high temperatures and high strain rates, Journal of Nuclear Materials 460 (2015) 184–199.
- [16] M. Salvo, J. Sercombe, J.-C. Ménard, J. Julien, T. Helfer, T. Désoyer,
 Experimental characterization and modelling of UO₂ behavior at high
 temperatures and high strain rates, Journal of Nuclear Materials 456
 (2015) 54–67.
- [17] R. F. Canon, J. T. A. Roberts, R. J. Beals, Deformation of UO₂ at High
 Temperatures, Journal of The American Ceramic Society 54 (1971) 105–
 112.
- 515 [18] A. G. Evans, R. W. Davidge, The strength and fracture of stoichiometric polycrystalline UO₂, Journal of Nuclear Materials 33 (1969) 249–260.

- [19] J. Mazars, F. Hamon, S. Grange, A new 3D damage model for concrete under monotonic, cyclic and dynamic loadings, Materials and Structures 48 (2015) 3779–3793.
- [20] J. Mazars, A description of micro- and macroscale damage of concrete structures, Engineering Fracture Mechanics 25 (1986) 729–739.
- 522 [21] B. Michel, J. Sercombe, G. Thouvenin, R. Chatelet, 3D fuel cracking 523 modelling in pellet cladding mechanical interaction, Engineering Frac-524 ture Mechanics 75 (2008) 3581–3598.
- [22] T. Helfer, Castelier, P. Garcia, Two dimensional modelling of PWR
 fuel rods analysis of fuel cracking, in: Euromech Mecamat 2006 :
 9th European Mechanics of Materials Conference Local Approach to
 Fracture.
- ⁵²⁹ [23] J. Byron, Yield and flow of polycrystalline uranium dioxide, Journal of Nuclear Materials 27 (1968) 48–53.
- 531 [24] Y. Guerin, Etude par compression a hautes temperatures de la deforma-532 tion plastique du bioxyde d'uranium polycristallin, Journal of Nuclear 533 Materials 56 (1975) 61–75.
- ⁵³⁴ [25] T. Helfer, B. Michel, J.-M. Proix, M. Salvo, J. Sercombe, M. Casella, Introducing the open-source mfront code generator: Application to mechanical behaviours and material knowledge management within the PLEIADES fuel element modelling platform, Computers & Mathematics with Applications 70 (2015) 994–1023.
- [26] N. Igata, K. Domoto, Fracture stress and elastic modulus of uranium dioxide including excess oxygen, Journal of Nuclear Materials 45 (1973)
 317–322.
- 542 [27] F. Sauter, S. Leclercq, Modeling of the non-monotonous viscoplastic 543 behavior of uranium dioxide, Journal of Nuclear Materials 322 (2003) 544 1–14.
- ⁵⁴⁵ [28] Cast3m 2019, http://www-cast3m.cea.fr/, 2019.
- [29] B. Michel, C. Nonon, J. Sercombe, F. Michel, V. Marelle, Simulation of
 pellet-cladding interaction with the PLEIADES fuel performance soft ware environment, Nuclear Technology 182 (2013) 124–137.

- [30] J.K. Fink, Thermophysical properties of uranium dioxide, Journal of
 Nuclear Materials 279 (1999) 1–18.
- [31] Hagrman, D T, Allison, C M, Berna, G A, SCDAP/RELAP5/MOD 3.1
 code manual: MATPRO, A library of materials properties for Light-Water-Reactor accident analysis, 1995.
- [32] I. A. E. Agency, Thermophysical properties database of materials for light water reactors and heavy water reactors: final report of a coordinated research project, 1999-2005., International Atomic Energy Agency, 2006.
- [33] D. Martin, The thermal expansion of solid UO₂ and (U, Pu) mixed
 oxides A review and recommendations, Journal of Nuclear Materials
 152 (1988) 94–101.
- [34] D. Martin, The elastic constants of polycrystalline UO₂ and (U, Pu)
 mixed oxides: A review and recommendations, High Temperatures.
 High Pressures 21.1 (1989) 13–24.
- [35] A. M. Wittenberg, Total Hemispherical Emissivity of Sapphire, Journal
 of the Optical Society of America 55 (1965) 432.