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# Assessment of Plutonium Inventory Management in the French Nuclear Fleet with the Fuel Cycle Simulator CLASS

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## Abstract

The ASTRID French project aimed at designing, building and operating a sodium fast reactor cooled with liquid sodium. One of the goals of the project was to demonstrate the feasibility of the plutonium multi-recycling in a fast spectrum. Commissariat à l'énergie atomique et aux énergies alternatives (CEA - Atomic & Alternative Energies Commission) has recently announced the abandon of the project which involves that no sodium fast reactor project is planned in France. As a consequence, there is a real interest in assessing technical feasibility for alternative plutonium management. In this work, the plutonium multi-recycling in PWR is assessed from fuel cycle simulations performed with the library CLASS developed by CNRS/IN2P3. The technical conditions for plutonium incineration and stabilization are investigated. It is shown that plutonium can be stabilized with 30% of PWR using multi-reprocessed plutonium in MOX fuel, the rest being composed by PWR loaded with UOX. The transuranic (plutonium and minor actinides) stabilization involves a plutonium incineration. For this reason, around 50% of PWR using multi-reprocessed plutonium is required and the nuclear power has to decrease. In this paper, the methodology and the output analysis are described in detail.

**Keywords:** Fuel Cycle Simulation, Plutonium reprocessing, Pressurized Water Reactors, Sensitivity analysis

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## **Acronyms**

**APA** Advanced Plutonium Assembly. 6

**BoC** Beginning of Cycle. 18, 19, 27

**BU** Burn up. 8, 10, 11, 13, 17, 18, 23, 27

30 **CORAIL** Combustible Recyclage A Ilot. 6

**DOE** Design Of Experiment. 8, 13, 14

**EoS** End of Scenario. 14–16, 18–20, 23–25

**FCS** Fuel Cycle Simulator. 10

**FP** Fission Products. 4, 5, 10

35 **FR** Fast Reactor. 4, 5

**HN** Heavy Nuclides. 10

**MA** Minor Actinides. 4, 5, 9, 10, 14–16, 18, 23–27, 29, 30

**MOX** Mixed OXide. 4, 5, 8–11, 31

**MOX-EUS** Mixed OXide on Enriched Uranium Support. 6, 7, 9–14, 16–19, 21–27,  
40 30–32

**PWR** Pressurized Water Reactor. 4–8, 10–14, 17–19, 21–27, 30–32

**SFR** Sodium Fast Reactor. 4, 5

**UOX** Uranium OXide. 4, 8–11, 13, 17, 18, 21–24, 26, 27

## 1. Introduction

45 Several countries are engaged in an energy transition in order to reduce greenhouse gases generated by fossil-fuel consumption. France is part of them and has definitely adopted the energy and climate bill in 2019 [1]. Main targets are mentioned below :

- Achieve zero net emissions and carbon neutrality by 2050
- 40% reduction of fossil fuel consumption by 2030 in comparison with 1990 level
- 50 • Reduce the share of nuclear energy in electricity at 50% by 2035

The French nuclear fleet is currently composed by 58 Pressurized Water Reactor (PWR)s fueled with Uranium OXide (UOX) and/or Mixed OXide (MOX). In a PWR fueled with UOX operated up to 40 GWd/t , the natural uranium consumption is around 200 tons/(GW<sub>elec</sub> · y) and the fraction of the initial mass that has fissioned is close to 55 4%. The same amount of energy can be obtained from the complete fission of 1.2 tons of uranium. In theory, it is possible to convert all the uranium into fissions but that would require plutonium breeding and multi-recycling of uranium and plutonium [2]. Fast Reactor (FR)s are usually supposed to perform more efficiently plutonium breeding compared to thermal spectrum reactors [3].

60 Important research programs dedicated to fast spectrum reactors were led since the 60's [4] with the goal of closing the fuel cycle. In France at that time, the context of uncertainty related to uranium availability required a more efficient utilization of the resource. In order to study this approach, three Sodium Fast Reactor (SFR)s have been designed, built and operated in France: Rapsodie [5], Phoenix [6] and Superphenix [7].

65 In parallel, a nuclear fuel reprocessing plant (La Hague Nuclear Recycling and Reprocessing Plant) and a plutonium-based fuel fabrication plant (Melox recycling plant, located in Marcoule) were built to produce MOX fuel used in PWR, intermediate steps before the nuclear fuel cycle closure. In this French very specific framework, the plutonium is a resource that is used to operate PWR and that may be used later to 70 load SFR. Ultimate waste would be composed by Minor Actinides (MA) and Fission Products (FP).

This long-term vision has faced many oppositions and SFR future after Superphenix final shutdown in 1997 was not clear. On 5 January 2006, the French president J. Chirac has announced the commissioning of a SFR demonstrator for 2020. This statement has  
75 been reinforced by the 2006 program act on the sustainable management of radioactive materials and wastes [8]. This has marked the beginning of the ASTRID program [9]. In this new context, the interest for SFR was also reinforced by their ability to transmute MA [10]. France's Commissariat à l'énergie atomique et aux énergies alternatives (CEA - Atomic & Alternative Energies Commission) has recently announced the abandon of  
80 the project [11]. Furthermore, the French Multi-annual Energy Programme (PPE) for the period 2019 to 2028 raises the construction of a SFR demonstrator at the end of the 21st century.

In this new context, the plutonium status could be redefined, from recoverable material to high-level waste. In 1991, the Law no. 91-1381 of 30 December 1991,  
85 known as the "Bataille" Law, defined three study axes for the high-level waste (MA and FP) [12]:

- Separation and Transmutation of long-life radioactive elements,
- Options for deep geological disposal,
- Packaging and long-term surface storage.

90 According to French Environment Code, radioactive waste is a radioactive substance for which no further use is planned or envisaged. This definition clearly excludes plutonium from nuclear waste categories since plutonium is currently used in PWR as MOX fuel and since plutonium incineration in SFR was planned. Plutonium inventory is increasing and the stockpiled mass in the current French fuel cycle is around 350  
95 tons [13].

Without any FR project, the plutonium reprocessing interest is not clear since gains, in terms of uranium resource or plutonium incineration, are relatively limited. In those conditions, there is a real interest in assessing technical feasibility for alternative plutonium management. This paper aims to study the plutonium multi-recycling in  
100 PWR from fuel cycle simulations with the code CLASS [14]. The technical conditions

for plutonium incineration and stabilization will be investigated. Tools and methodology will be presented. Outputs analysis will lead to fuel cycle parameters that fit with those objectives.

## 2. Plutonium multi-recycling in PWR

### 105 2.1. Physics of Pu multi-recycling in PWR

Plutonium multi-recycling in PWR is a technological challenge, mainly because of safety issues. Plutonium irradiation in a thermal spectrum induces a relative decrease of odd fissile isotopes ( $^{239}\text{Pu}$  and  $^{241}\text{Pu}$ ) compared to even isotopes ( $^{238}\text{Pu}$ ,  $^{240}\text{Pu}$  and  $^{242}\text{Pu}$ ). As a consequence, the plutonium fraction in the fresh fuel needed to reach  
110 criticality during a reactor cycle increases. Nevertheless, increasing the plutonium fraction may lead to a deterioration of the void coefficient. In practice, these maximum calculated contents vary from 12% to 15% according to the plutonium isotopic composition [15, 16, 17].

Innovative designs have been studied in order to overcome those issues [18]. Some  
115 relevant and advanced concepts are based on uranium and plutonium elements and are listed below:

- Mixed OXide on Enriched Uranium Support (MOX-EUS) also called MIX concept,
- Combustible Recyclage A Ilot (CORAIL),
- 120 • Advanced Plutonium Assembly (APA),
- DUPLEX,
- REMIX technology.

Those concepts are based on the use of  $^{235}\text{U}$  to compensate for the lack of reactivity that results from the constraint on maximal plutonium fraction imposed in fresh fuel. A  
125 comparative study of those assembly concepts can be found on [19, 20]. In Russia, an innovating fuel fabrication technology called REMIX-technology aims to build a fuel

made from mixtures of uranium and plutonium isotopes blended with enriched natural uranium [21, 22]. A recent work presents characteristics of REMIX fuel assembly [23].

## 2.2. Description and simulation of MOX on enriched uranium support concept

130 In the framework of this work, the MOX-EUS fuel has been investigated because of the concept simplicity and its high plutonium incineration capacity. The MOX-EUS concept is a standard PWR assembly composed by 264 homogeneous pins of plutonium and uranium mix.

135 Reactor models have been built and integrated in CLASS for PWR fueled with MOX-EUS from Monte-Carlo assembly calculations. A databank composed of 1000 PWRs infinite assembly (see Figure 1 for a view of the assembly section) depletion calculations performed using the software MURE (MCNP Utility for Reactor Evolution) [24] based on the transport code MCNP (Monte-Carlo N Particle) [25].

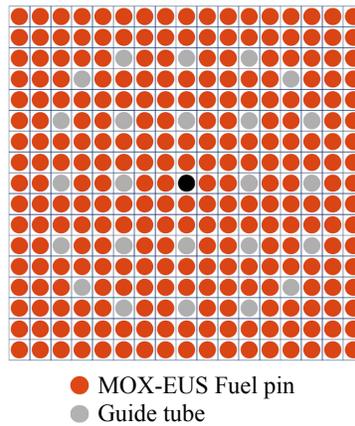


Figure 1: Evolution of the effective thermal power of the simulated French nuclear fleet (black, left vertical axis). Evolution of the plutonium (green, right vertical axis) and minor actinides mass (dotted line, green, right vertical axis).

140 Neural networks are trained to reproduce neutronic outputs with an excellent accuracy. Validated neural networks are used to build the fresh fuel and to calculate the fuel evolution during irradiation. This preliminary work has been fully described in [26].

### 3. Scenario study description

In this section, the scenario Design Of Experiment (DOE) is presented. All the fuel cycle simulations performed in this work are done with the code CLASS (Core Library  
145 for Advanced Scenarios Simulations) [13]. The precise definition of the initial time for the simulations is also described.

#### 3.1. Scenario initial condition

A precise simulation of the French nuclear fleet has been performed, from the beginning of the nuclear program up to 2015 which is the starting point of scenario [27].  
150 In order to have a precise simulation, input data (reactor Burn up (BU), load factors, cooling time, etc.) need to be robust and reliable. Due to sensitivity of this kind of information, collecting quality data may be complex. As a consequence, an important literature review has been performed in order to extract, compare and sort available and public information. References used in the framework of this work to reconstruct the  
155 French nuclear history are specified in the annex B of [27].

The French nuclear fuel cycle simulation takes into account 58 PWRs. These PWRs are classified according to their nominal thermal power  $P_{th}$  and heavy nuclides mass  $M_{HN}$  in the core into different categories called series:

- 34 PWRs 900 ( $P_{th} = 2.8 \text{ GW}_{th}$ ,  $M_{HN} = 72.5 \text{ tons}$ )
- 160 • 20 PWRs 1300 ( $P_{th} = 3.8 \text{ GW}_{th}$ ,  $M_{HN} = 104 \text{ tons}$ )
- 4 PWRs 1450 ( $P_{th} = 4.3 \text{ GW}_{th}$ ,  $M_{HN} = 110 \text{ tons}$ )

In 2015, 22 PWRs 900 are loaded with around 30% of MOX fuel since other reactors are loaded with UOX fuel. Several industrial fuel management strategies characterized by a different BU, fissile enrichment or fuel loading plan are used. A reactor can also  
165 switch from one management to another during its lifetime. Reactors fuel discharge BU or industrial fuel management change dates extracted from public literature vary from one source to another. Reference data used to reconstruct the French nuclear fleet are then based on average of available reliable data.

Reactors load factors are calculated from the PRIS (Power Reactor Information  
 170 System) database [28]. Annual load factors per reactor are summarized through an  
 averaged value that is used for all the reactor lifetime. The thermal power used in the  
 simulation, called effective thermal power, is then the nominal thermal power multiplied  
 by the averaged load factor. The Figure 2 shows the effective simulated thermal power  
 and the plutonium and MA total mass evolution from the beginning of the French  
 175 nuclear program up to 2015. Spent fuel and waste inventories computed from this very  
 detailed simulation are used as the initial conditions for scenarios.

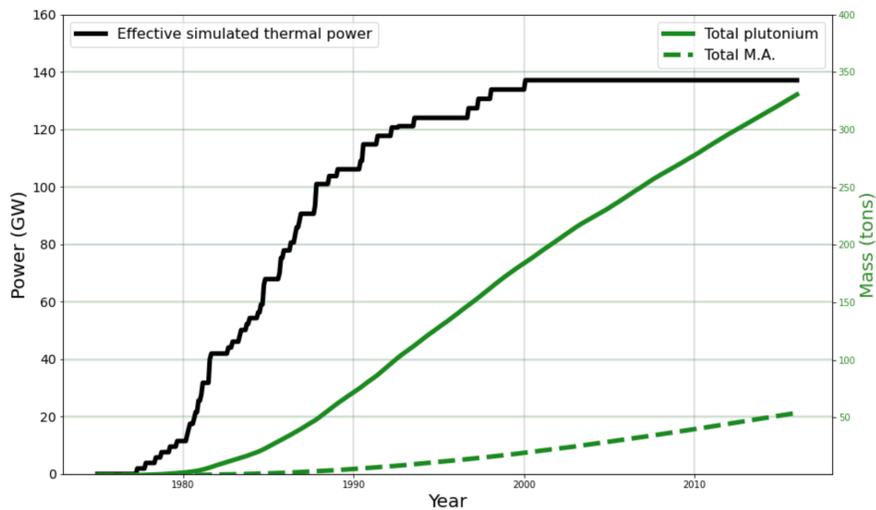


Figure 2: Evolution of the effective thermal power of the simulated French nuclear fleet (black, left vertical axis). Evolution of the plutonium (green, right vertical axis) and minor actinides mass (dotted line, green, right axis).

### 3.2. Scenario framework

In this work, a large sampling methodology on input parameters has been performed. In order to simulate a high number of trajectories, macro reactor approach has been  
 180 used. Each reactor are combined and gathered into a unique reactor by type of fuel  
 in the problem: UOX, MOX and MOX-EUS. Reactors parameters evolution can be  
 treated as quasi-continuous. The annual power is modified from the fuel mass update  
 by considering a constant specific power. This approach highly reduces the complexity

of the simulation, the size of outputs and the simulation time. The viability of this  
185 important approximation has been validated and discussed in [27].

The scenario representation is shown on the Figure 3. The fleet is composed of  
two macro PWRs fueled with UOX (green facilities) and MOX (blue facilities). The  
transition toward the plutonium multi-reprocessing strategy consists in replacing the  
MOX component by the MOX-EUS stratum (red facilities). The fuel cycle starts with  
190 an infinite stock of natural uranium  $U_{\text{nat}}$  used to build enriched Uranium  $U_{\text{enr}}$  in the  
UOX fabrication plant. UOX enrichment is calculated according to the PWR BU. After  
one irradiation cycle, the spent UOX fuel is cooled few years in a pool and sent in a  
stock that contains at this step Heavy Nuclides (HN) (uranium MA and plutonium) and  
FP. This stock and an infinite stock of depleted uranium  $U_{\text{dep}}$  are used by the MOX  
195 fabrication plant to build MOX fuel. After one irradiation cycle in PWR MOX fuel,  
the spent fuel is cooled in a pool and sent in a stock. The MOX-EUS fuel is built from  
three streams: a plutonium stream extracted from spent fuels, a  $^{235}\text{U}$  stream from an  
infinite stock and a  $U_{\text{dep}}$  stream. The MOX-EUS fuel is charged in the reactor, sent  
to a cooling pool and sent back to the spent MOX stock in order to be reprocessed.  
200 Historical UOX and MOX spent fuel inventories produced by the French nuclear fleet  
up to 2015 are also introduced in this cycle. Two stocks are generated for this purpose  
and are composed by spent UOX and MOX in 2015. At each fresh fuel fabrication step,  
MA and FP are sent to a final stock and are not reused during the dynamic simulation.

In the framework of the design of experiments described above, trajectories simu-  
205 lated with the Fuel Cycle Simulator (FCS) CLASS starts in 2015. In order to have a  
fuel cycle description at this time which is representative of the French fleet, a detailed  
simulation has been done and has been described on the section 3.1. From this starting  
point, reactor power sampling set is represented on Figure 4.

The scenario starts at  $T_0 = 2015$  with a total effective thermal power  $P_0 = 137.13 \text{ GW}_{\text{th}}$   
210 which corresponds to a total installed thermal power of  $P_0 = 188.11 \text{ GW}_{\text{th}}$  with a load  
factor of 0.729. Scenario ends at  $T_{\text{final}} = 2115$  with an effective thermal power of  $x_p P_0$ .  
Nevertheless, for output representation, the time will be labeled from 0 to 100 years.  
The thermal power decrease starts at the year  $B_D = T_0 + 10\text{y}$  and the decrease slope  
is constant up to the year  $E_D$ . MOX-EUS fuel is deployed instantly in PWR at the

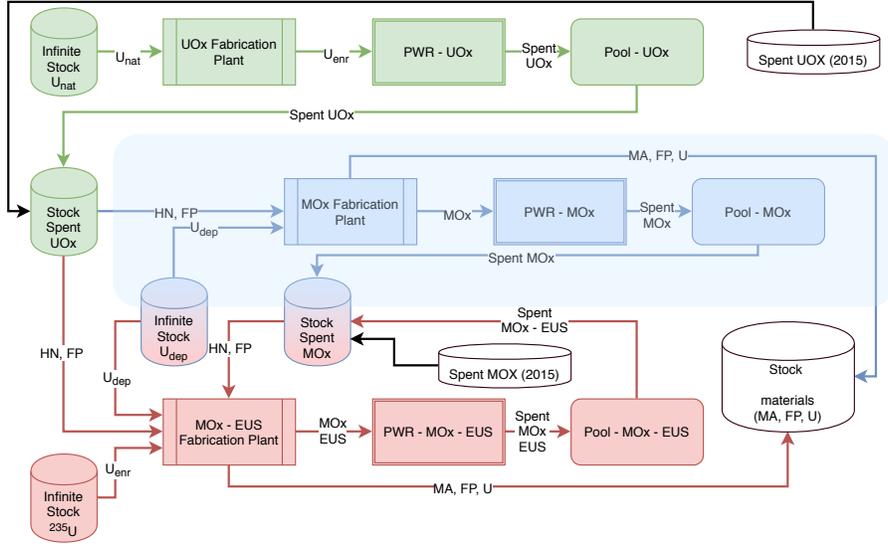


Figure 3: Schematic view of the fuel cycle simulations after 2015. Green facilities represent PWR UOX strata. Blue facilities represent PWR MOX components. The red part of the scheme is the PWR MOX-EUS that closes the fuel cycle for plutonium. Spent UOX and MOX in black are the homogeneous stocks obtained by the detailed French simulation until 2015.

215 year  $B_E = T_0 + 20y$ . The fraction of PWR fueled with MOX-EUS power on the total thermal power is  $x_E$ . At this end of the scenario, after 100 years, the effective thermal power for PWR fueled with MOX-EUS is  $x_E x_P P_0$ . Only a decrease of thermal power is supposed in order to be in agreement with the maximal capacity production of 63.2 GW imposed by the National Energy Code [29]. All fuel fabrication times are set to 2 years.

220 Plutonium separation efficiency is 100%. The parameter for reactor models  $k_{\text{threshold}}$  has been set to 1.037 with 4 fuel loading batches. Load factors for reactors have been set to 0.73. Before the time  $B_E$  and during the power decrease, the MOX fraction, the BU and the MOX spent fuel cooling time are respectively 10%, 45 GWd/t and 8 years.

### 3.3. Input variable space

225 Variables and sampled technical parameters of the simulation have been reported in the Table 1. Those constraints have been defined according to technical hypotheses related to the future of the nuclear fleet. Reactor BU are typical PWR ranges. The

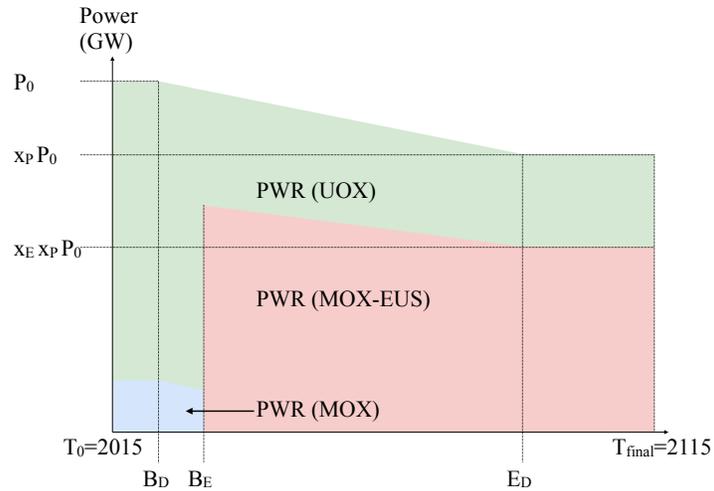


Figure 4: Trajectories space for the effective thermal power evolution of PWR fueled with UOX, MOX and MOX-EUS from 2015 to 2115.

fraction of the thermal power based on PWR MOX-EUS fuel is completely open, between 0 and 100%.

230 The maximum plutonium content allowed in a MOX-EUS assembly is strongly dependent on the plutonium isotopic composition and fixing a precise value requires further system studies. To take into account safety aspects related to high plutonium content in the fuel, a plutonium maximum content range has been defined, between 8% and 13%. This maximum plutonium content range is in agreement with typical values  
 235 used in nuclear scenario studies [30]

Spent fuel cooling time can have an impact on material availability and composition.  $E_D$  and  $x_P$  range has been chosen in order to include a wide range of possibility for the evolution of the nuclear fleet. The plutonium stock management has been sampled on two values: LiFo (Last in First out) and FiFo (First in First out). In the LiFo management,  
 240 the most recent plutonium is taken from the stock. In the FiFo management, the oldest plutonium in the stock is taken first.

Table 1: Ranges used for sampled parameters used in CLASS simulations.

Name	Sampled parameter	Range
$BU_U$	PWR UOX BU	[30, 65] GWd/t
$BU_E$	PWR MOX-EUS BU	[30, 65] GWd/t
$x_E$	PWR MOX-EUS fuel thermal power fraction	[0, 1]
$w_{Pu}^{MAX}$	Maximum Pu content in the MOX-EUS	[0.8, 0.13]
$T_c^U$	UOX fuel cooling time	[3, 10] years
$T_c^E$	MOX-EUS fuel cooling time	[3, 10] years
$E_D$	End of the power decrease	[20, 100] years
$x_P$	Ratio between final and initial thermal power	[0.01, 1.0]
$G_s$	Plutonium stock management	LiFo, FiFo

### 3.4. Design Of Experiments

From this set of fixed and variable parameters, two DOE have been used. The first one is called  $DOE_{Sobol}$  and has been used to calculate Sobol' indices.  $DOE_{Sobol}$  is built from the method *SobolSalt* from the package *Sensitivity* [31] of the software environment for statistical computing and graphics R [32]. Two input samples have been built from a Monte-Carlo method. The total number of required runs is  $M \times (2N + 2)$  where  $N$  is the number of quantitative input variables (8 in this case) and  $M$  is the number of runs in each sample.  $M$  has been fixed at 2000, which imposes 36000 CLASS code callings, for each plutonium stock management. From this DOE, total, first and second order indices have been calculated for each input variables [33, 34]. The total Sobol' index represents the contribution of one input variable in the output variance. First order Sobol' index represents the contribution of one input variable alone in the output variance, while the second order index represents the contribution of the interaction of two input variables on the output variance.

The second one,  $DOE_{LHS}$  is based on a Latin Hyper Square sampling on quantitative variables described in the Table 1. This DOE is composed by 10000 CLASS simulations

for each value of  $G_s$ , LiFo and LiFo. This DOE is used to analyze output variable of interest described in the next section.

260 A total of 92000 CLASS runs have been performed ( $2 \times 36000 + 2 \times 10000$ ) on the IN2P3<sup>1</sup> computing centre batch farm. The order of magnitude of the time required to run all these simulations is around the day.

### 3.5. Output and nomenclature

Each CLASS simulation produces a ROOT [35] output file that stores the simulation parameters and results. For each DOE, a global ROOT file is generated in order to gather 265 results of interest. The total size of output data is around 20 Go. Structure of output files is composed by a list of events. Each event represents a CLASS simulation and contains the following stored data:

- Input parameters value
- 270 • Effective thermal power and cumulated energy evolution
- Transuranic mass evolution in facilities

The nomenclature for element mass consists in writing element in subscript and location in superscript. Elements can be U, Pu or any MA (Np, Am or Cm). Location can be any facility, stocks, pools and waste. Total represents the sum in the fleet. For 275 example, the plutonium and MA mass evolution in the PWR MOX-EUS is  $M_{\text{Pu+MA}}^{\text{MOX-EUS}}(t)$ . The total plutonium mass in the fleet at this End of Scenario (EoS) is called  $M_{\text{Pu}}^{\text{tot}}(\text{EoS})$ .

## 4. Scenario Output analysis

This section aims to present raw output results in order to clarify and understand the simulation set.

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280 **4.1. Total power, energy and inventory**

Figure 5 shows evolution for the effective thermal power, the cumulated energy, the plutonium and MA masses evolution for the FiFo data set. Three random examples have been selected in order to highlight some trajectories.

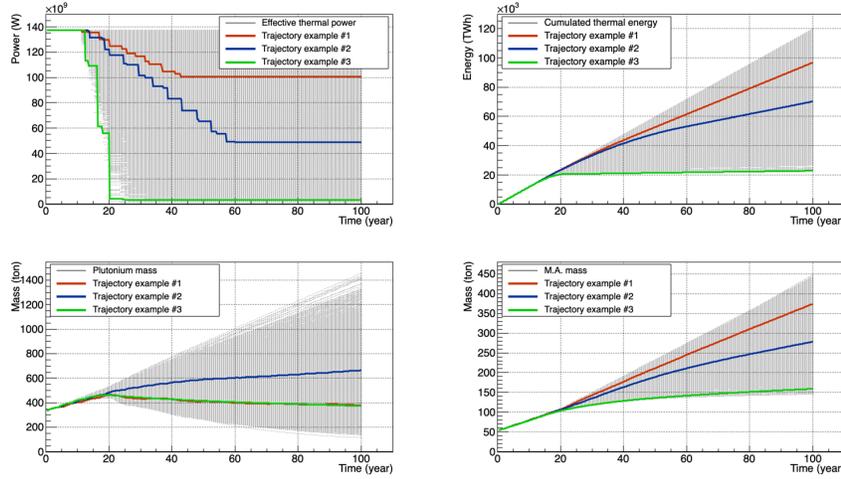


Figure 5: Effective thermal power (top left plot) evolution, cumulated energy production evolution (top right plot), total plutonium mass evolution (bottom left) and total minor actinides (bottom right) mass evolution for the FiFo set of data. Three random trajectory examples have been represented.

The Figure 5 shows that thermal power and cumulated energy range is very wide, from the continuation of nuclear at constant power to a fast phase out. Plutonium mass evolution is also very diversified. Some trajectories show plutonium incineration while other produce up to 1400 tons at EoS. Since there is no MA reprocessing considered in this work, all calculated trajectories are MA producer. MA mass at EoS ranges between 150 and 450 tons.

290 **4.2. Sobol' indices analyse**

Sobol' indices have been calculated and following nomenclature have been used :

- $\widehat{S}_i$  : First order Sobol' indices related to variable  $i$
- $\widehat{S}_{ij}$  : Second order Sobol' indices related to the interaction of variable  $i$  with variable  $j$

- $\widehat{T}_i$  : Total Sobol' indices related to variable  $i$

The Table 2 presents Sobol' indices for the total plutonium mass at EoS ( $M_{\text{Pu}}^{\text{tot}}(\text{EoS})$ ) and for the total plutonium and MA mass at EoS ( $M_{\text{Pu+MA}}^{\text{tot}}(\text{EoS})$ ). FiFo and LiFo management strategies for plutonium stocks used to build the MOX-EUS fuel are separated. Sobol' indices represented in the table are selected according to the value and the error on the value. If the error is higher than the value, the indices is not represented. As a consequence, only total and first order indices are represented, which means that second order interaction between variables are negligible in this case.

Table 2: First order, second order and total Sobol' indices estimation for output variables  $M_{\text{Pu}}^{\text{tot}}(\text{EoS})$  and  $M_{\text{Pu+MA}}^{\text{tot}}(\text{EoS})$  at the end of the scenario.

Fuel Strategy	Indices	$M_{\text{Pu}}^{\text{tot}}(\text{EoS})$	$M_{\text{Pu+MA}}^{\text{tot}}(\text{EoS})$
		Index / Error	Index / Error
LiFo	$\widehat{S}_{\text{BU}_U}$	0.032 / 0.024	0.033 / 0.021
	$\widehat{S}_{\text{x}_E}$	0.825 / 0.005	0.648 / 0.013
	$\widehat{S}_{\text{x}_P}$	0.097 / 0.024	0.260 / 0.019
	$\widehat{T}_{\text{BU}_U}$	0.032 / 0.001	0.032 / 0.001
	$\widehat{T}_{\text{x}_E}$	0.875 / 0.024	0.691 / 0.017
	$\widehat{T}_{\text{E}_D}$	0.014 / 0.001	0.030 / 0.001
	$\widehat{T}_{\text{x}_P}$	0.131 / 0.005	0.300 / 0.010
	FiFo	$\widehat{S}_{\text{BU}_U}$	0.036 / 0.021
$\widehat{S}_{\text{x}_E}$		0.820 / 0.007	0.650 / 0.012
$\widehat{S}_{\text{x}_P}$		0.110 / 0.021	0.260 / 0.018
$\widehat{T}_{\text{BU}_U}$		0.033 / 0.001	0.032 / 0.002
$\widehat{T}_{\text{x}_E}$		0.860 / 0.022	0.690 / 0.017
$\widehat{T}_{\text{E}_D}$		0.013 / 0.001	0.030 / 0.002
$\widehat{T}_{\text{x}_P}$		0.140 / 0.005	0.300 / 0.001

Sobol' indices for LiFo and FiFo plutonium management strategy are compatible within the errors. Four variables are represented :  $BU_U$ ,  $x_E$ ,  $E_D$ ,  $x_P$ . The fraction of PWR MOX-EUS  $x_E$  is by far the most important variable involved in the variance of the outputs  $M_{Pu}^{tot}(EoS)$  and  $M_{Pu+MA}^{tot}(EoS)$  since total and first order Sobol' indices are between 80% and 90%. The ratio between final and initial thermal power  $x_P$  has a smaller impact on outputs variance with a total and first order Sobol' indices close to 10%. The BU of the PWR UOX has a limited effect that is quantified by Sobol' indices around 3%, very close to the statistical errors. The time at which the power ends decreasing ( $E_D$ ) presents a small value of total Sobol' indices and negligible for the first and second order indices.

The role of  $x_E$  in the output variance is quite clear. All other factors being equal, the higher  $x_E$  is, the higher the plutonium consumption rate is and the smaller plutonium inventory is at the end of the scenario. This can be confirmed by the plot represented on Figure 6 which represents  $M_{Pu}^{tot}(EoS)$  and  $M_{Pu+MA}^{tot}(EoS)$  according to  $x_E$  and  $x_P$  for the LiFo set of data.

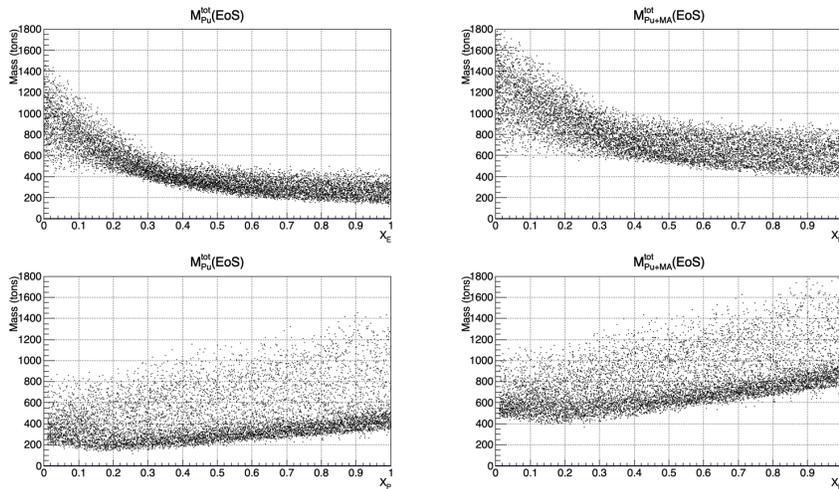


Figure 6:  $M_{Pu}^{tot}(EoS)$  and  $M_{Pu+MA}^{tot}(EoS)$  dependency according to  $x_E$  for the LiFo Design Of Experiment (first row).  $M_{Pu}^{tot}(EoS)$  and  $M_{Pu+MA}^{tot}(EoS)$  dependency according to  $x_P$  for the LiFo Design Of Experiment (second row).

The impact of  $x_P$  is more difficult to assess. If  $x_P$  is very small, that means the fleet

thermal power tends to zero which is a nuclear phase out during the scenario. A phase  
320 out means the plutonium inventory is slightly decreasing (due to  $^{241}\text{Pu}$  radioactive decay  
into  $^{241}\text{Am}$ ) while the total inventory of plutonium and MA is constant. If  $x_P$  increases,  
the impact on plutonium and MA inventory depends on those elements balance in the  
fleet. If the fleet produces Pu (respectively Pu + MA) in average, the final inventory of  
Pu (respectively Pu + MA) will be higher. On the other hand, if the fleet is Pu incinerator  
325 in average, the final inventory of Pu will be smaller. In this case, the final balance of  
the sum of Pu and MA masses will depend on the decrease rate of the plutonium, in  
comparison with MA production.

Representations of  $M_{\text{Pu}}^{\text{tot}}(\text{EoS})$  and  $M_{\text{Pu+MA}}^{\text{tot}}(\text{EoS})$  according to other input variables  
can be found in [27]. For those other representations, dependencies are negligible.

#### 330 4.3. Outputs of interest

Outputs of interest are presented in the Table 3. The first family is composed by  
output built from total plutonium mass. Three estimators have been built. The first one  
is the mass balance between EoS and the deployment time of PWR MOX-EUS. The  
second one is the mass balance between EoS and  $t=60$  years. The third one is the mass  
335 balance between 60 years and the deployment time of PWR MOX-EUS. Using those  
three estimators aims to remove bell curve trajectories for which the inventory balance  
between EoS and  $t=20$  years would be close to zero. The second family of output of  
interest is similar but concerns the sum of plutonium and MA.

Some trajectories may have a bias related to the value of  $x_E$ . A lack of plutonium  
340 does not prevent the start of a PWR MOX-EUS that can be operated with a high  $^{235}\text{U}$   
enrichment. In those conditions, such a reactor should be considered as a PWR UOX  
even though it is recorded as a PWR MOX-EUS. In order to discriminate this kind  
of behavior, an additional output,  $\langle \text{U5 Enr.} \rangle$  has been produced and represents the  
average uranium enrichment in fresh fuel at each reactor loading in the given trajectory.  
345 We call NL the number of fresh fuel loadings during a trajectory. As the final time is  
fixed, NL depends on the reactor cycle time which is calculated from the reactor BU.  
The ratio between the  $^{235}\text{U}$  mass at Beginning of Cycle (BoC) ( $M_{235\text{U}}^{\text{MOXEUS}}(\text{BOC})$ ) in  
the PWR MOX-EUS and the heavy nuclides fuel mass ( $M_{\text{fuel}}$ ) represents the fresh fuel

<sup>235</sup>U enrichment. This output variable will be used to control the PWR MOX-EUS characteristics.

This will be completed by the average plutonium content at BoC in a trajectory calculated as the ratio of the plutonium mass in the reactor BoC ( $M_{Pu}^{MOX-EUS}(BoC)$ ) and the heavy nuclides fuel mass ( $M_{fuel}$ ).

Finally, the cumulated thermal energy at EoS ( $E_{th}(EoS)$ ) has been calculated for each trajectory as the thermal power integration on time. This output will provide additional information on the nuclear power evolution.

Table 3: Output parameters of interest for the analyze of simulation data set. Sums are done on all reactor loadings in the scenario.

Label	Definition	Unit
$\Delta M_{Pu}^{tot}(A)$	$M_{Pu}^{tot}(EoS) - M_{Pu}^{tot}(20y)$	tons
$\Delta M_{Pu}^{tot}(B)$	$M_{Pu}^{tot}(EoS) - M_{Pu}^{tot}(60y)$	tons
$\Delta M_{Pu}^{tot}(C)$	$M_{Pu}^{tot}(60y) - M_{Pu}^{tot}(20y)$	tons
$\Delta M_{Pu+MA}^{tot}(A)$	$M_{Pu+MA}^{tot}(EoS) - M_{Pu+MA}^{tot}(20y)$	tons
$\Delta M_{Pu+MA}^{tot}(B)$	$M_{Pu+MA}^{tot}(EoS) - M_{Pu+MA}^{tot}(60y)$	tons
$\Delta M_{Pu+MA}^{tot}(C)$	$M_{Pu+MA}^{tot}(60y) - M_{Pu+MA}^{tot}(20y)$	tons
< U5 Enr. >	$\frac{\sum_{i=1}^{i=NL} M_{U5}^{MOX-EUS}(BOC_i)}{M_{fuel}}$	-
< Pu Frac. >	$\frac{\sum_{i=1}^{i=NL} M_{Pu}^{MOX-EUS}(BoC)}{NL}$	-
$\Delta E_{th}$	$\int_{t=20y}^{t=EoS} P_{th}(t)dt$	TWh

No important differences have been observed between LiFo and FiFo strategies for plutonium stock management. As a consequence, all following results will be presented for the FiFo data set.

## 5. Plutonium stabilization

In this section, the stabilization of the plutonium inventory will be investigated.

### 5.1. stabilization criteria

The plutonium stabilization condition is based on the output observable  $\Delta M_{\text{Pu}}^{\text{tot}}(\text{A})$ ,  $\Delta M_{\text{Pu}}^{\text{tot}}(\text{B})$  and  $\Delta M_{\text{Pu}}^{\text{tot}}(\text{C})$ . The plutonium stabilization criteria are following :

$$-20 \text{ tons} < \Delta M_{\text{Pu}}^{\text{tot}}(\text{A, B, C}) < 20 \text{ tons} \quad (1)$$

365 A  $\pm 20$  tons deviation is allowed on a plutonium mass. Using two more criteria by splitting trajectories aims to remove bell curve trajectories that would respect  $\Delta M_{\text{Pu}}^{\text{tot}}(\text{A})$  criterion by accident. The Figure 7 shows plutonium evolution for the FiFo data set and for the plutonium stabilization selection obtained by application of criteria described above.

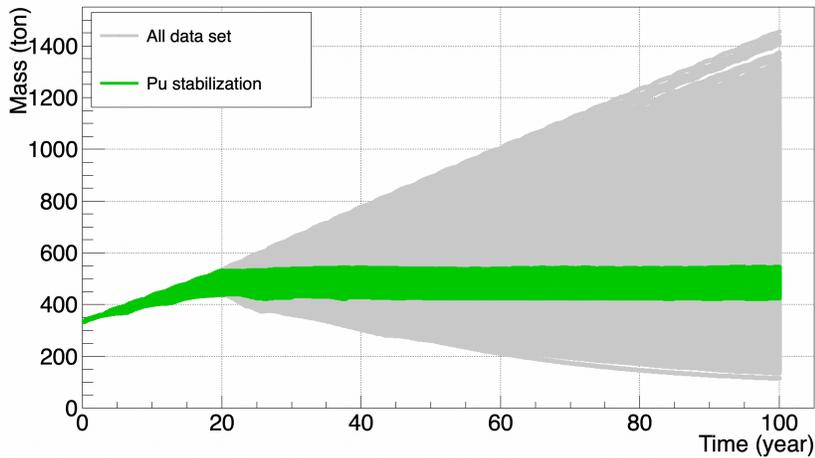


Figure 7: Total plutonium evolution for the FiFo data set (gray lines). Trajectories obtained after plutonium stabilization criteria are highlighted in green.

370 The plutonium mass evolution is constant as expected and ranges between 400 and 550 tons at EoS. In order to identify input variables space for which plutonium is stabilized, parallel plots are generated and shown on Figure 8. Three cuts have been imposed on  $\langle \text{Pu Frac.} \rangle$  parameter for the readability. This highlights the fact that some stabilization trajectories are characterized by small plutonium content (blue selection on the Figure 8 and a high uranium enrichment. Those trajectories allow  
375

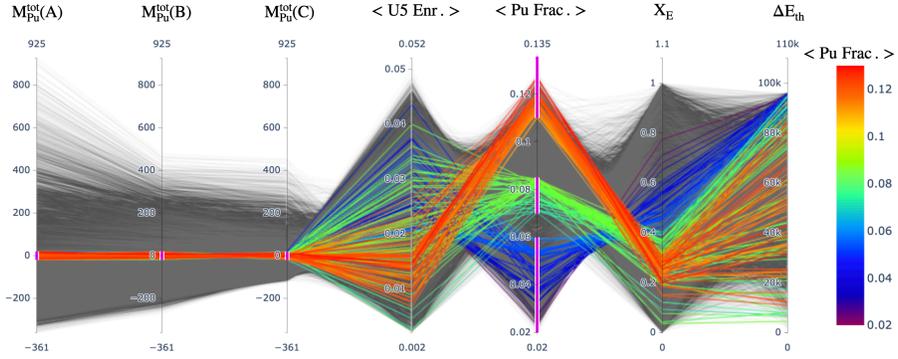


Figure 8: Parallel plot built from the FiFo data set. Gray lines represent all the simulations. Colored lines are trajectories that stabilize the total plutonium mass. Colored right scale is a color code based on the plutonium fraction at beginning of the reactor cycle averaged on each trajectory. Each vertical line is a variable defined in the text. For readability, three cuts have been imposed on  $\langle \text{Pu Frac.} \rangle$ .

higher PWR MOX-EUS fraction ( $x_E \sim 0.5$ ) with high values for  $x_P$ , which means almost constant power nuclear fleet. The higher value of  $x_E$  is a bias since PWR MOX-EUS behaves as a UOX. As a consequence, the plutonium stabilization input space assessment must consider only high plutonium content to have a non-biased value of  $x_E$ . as a consequence, following cut has been imposed on  $\langle \text{Pu Frac.} \rangle$  :

$$\langle \text{Pu Frac.} \rangle \geq 0.07 \quad (2)$$

Also, small values of  $x_E$  ( $x_E \leq 0.2$ ) leads to really low cumulated thermal energy. That means those trajectories represent the short term nuclear phase out in which plutonium is close to equilibrium because there are no reactors in operation. An additional cut ( $E_{th}(EoS) \geq 20 \cdot 10^E \text{ TWh}$ ) is then imposed.

## 5.2. Input variables distributions

The final distribution for  $x_E$  and interactions with important input data are shown on Figure 9. In the case of Pu stabilization,  $x_E$  is represented by a normal distribution, with a 4.5% standard deviation. The PWR MOX-EUS fraction that stabilizes the plutonium is around 30%. Nevertheless, this value is highly correlated with other input variables.  $x_E$  and  $x_P$  are positively correlated, which means that a higher fraction of PWR MOX-EUS

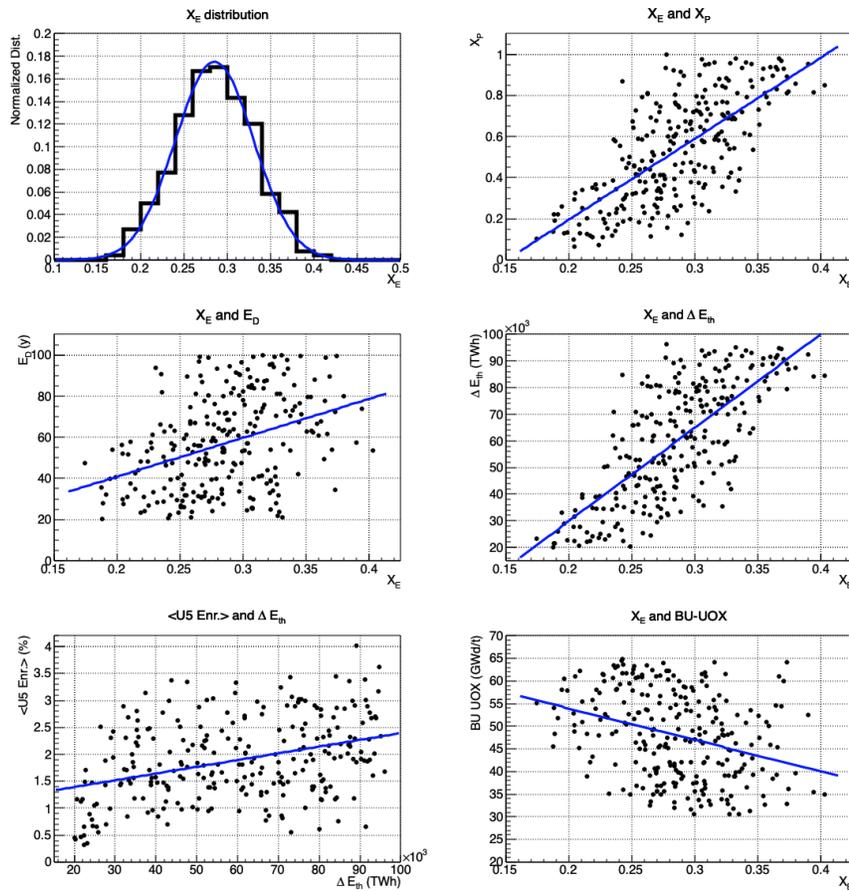


Figure 9:  $x_E$  final distribution for the FiFo data set induced by the plutonium stabilization (top left) fitted by a normal distribution. Interaction of  $x_E$  with  $x_P$  (top right),  $E_D$  (middle left),  $\Delta E_{th}$  (middle right) and BU UOX (bottom right) are represented. The bottom left plot represents the average enrichment in  $^{235}\text{U}$ ,  $\langle \text{U5 Enr.} \rangle$  versus  $\Delta E_{th}$ . First-order polynomial fitted functions are also represented in blue.

are required to stabilize the plutonium if the fleet power is kept high. The conclusion is equivalent when looking at  $\Delta E_{th}$  interaction with  $x_E$ . The role of  $E_D$  is much smaller. To explain these observations, we recall that the plutonium equilibrium comes from a competition between reactors that produce plutonium (PWRs UOX) and reactors that incinerate plutonium (PWRs MOX-EUS). If the nuclear power is kept high, plutonium mass in PWR MOX-EUS is high. To give an order of magnitude of plutonium mass involved, let consider the French fleet described above. The total heavy nuclides mass

in reactors,  $M_{\text{HN}}^{\text{Reac.}}$ , is given by :

$$M_{\text{HN}}^{\text{Reac.}} = 34 \times 75.5 + 20 \times 104 + 4 \times 110 = 587 \text{ tons} \quad (3)$$

Having 30% of the fleet dedicated to the multi-reprocessing of the plutonium with  
 400 reactors loaded with around 10% of plutonium means that around 150 tons is mobilized  
 in the PWR MOX-EUS. A rough factor 3 can be applied to take into account the interim  
 storage of the plutonium (2 years for fuel fabrication and 5 years for spent fuel cooling).  
 The final order of magnitude is around 450 tons, closed to the initial plutonium mass.  
 This means that a tightness on plutonium may occur for trajectories characterized by a  
 405 high power. As a consequence, high-power trajectories will need more enrichment in  
 $^{235}\text{U}$ . The plot representing  $\langle \text{U5 Enr.} \rangle$  as a function of  $\Delta E_{\text{th}}$  (third row, left) shows  
 that  $^{235}\text{U}$  enrichment doubles between small and high values of  $\Delta E_{\text{th}}$ .

The BU UOX interaction with  $x_E$  shows that a small value of BU required a high  
 value of  $x_E$  in order to stabilize the plutonium inventory. This can be interpreted by  
 410 the fact that the plutonium generation rate is higher for smaller BU and the plutonium  
 mass to manage is also higher. Statistical parameters of input variable distributions are  
 summarized in the Table 4.

Table 4: Statistical parameters for input variable distributions in the case of plutonium stabilization obtained from the FiFo data set.

Variable	$x_E$	$x_P$	$E_D$	BU UOX	$\Delta E_{\text{th}}$
mean	$2.9 \cdot 10^{-1}$	$5.3 \cdot 10^{-1}$	$5.7 \cdot 10^1$	$4.8 \cdot 10^1$	$6.0 \cdot 10^4$
std	$4.5 \cdot 10^{-2}$	$2.5 \cdot 10^{-1}$	$2.2 \cdot 10^1$	9.6	$2.2 \cdot 10^4$
min	$1.7 \cdot 10^{-1}$	$6.6 \cdot 10^{-2}$	$2.0 \cdot 10^1$	$3.1 \cdot 10^1$	$2.0 \cdot 10^4$
max	$4.0 \cdot 10^{-1}$	1.0	$1.0 \cdot 10^2$	$6.5 \cdot 10^1$	$9.6 \cdot 10^4$

### 5.3. Minor Actinides production

The Figure 10 represents the ratio between MA and plutonium mass at EoS. This  
 415 ratio ranges between 40% and 100%. The plot also shows that MA production in-  
 creases with  $x_E$ . This is explained by the fact that increasing the plutonium irradiation

produces more MA by neutron capture on plutonium. In addition, it shows that MA inventory order of magnitude is very comparable to plutonium inventory at EoS. As a consequence, the MA should be taken into account in the framework of high mass inventory stabilization. This is investigated in the following section.

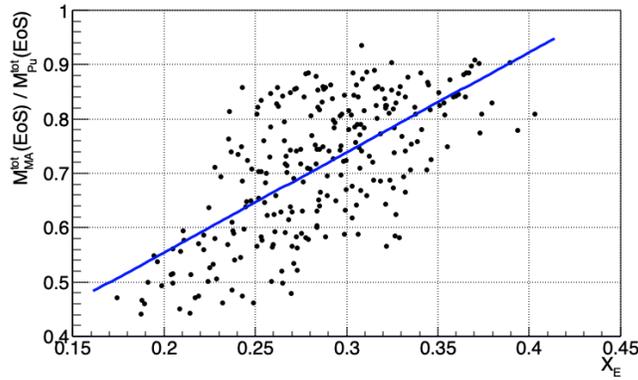


Figure 10: Ratio between minor actinides mass and plutonium mass at the end of the scenario as a function of  $x_E$ . First-order polynomial fitted functions are also represented in blue.

In order to have a global vision of trajectories that allow plutonium stabilization, the power evolution for the final subset of data is represented on Figure 11. Two extreme trajectories have been highlighted. The first one corresponds to the smallest value for  $\Delta E_{th}$  (red line) and the second one is the highest (blue line). Two random trajectories (green and pink lines) are also highlighted.

#### 5.4. Plutonium stabilization summary

To close this section related to the plutonium stabilization, the following summary is proposed. Among all the simulated trajectories involving multi-reprocessing of plutonium in PWR MOX-EUS, some of them show plutonium inventory stabilization. However, part of them includes PWR MOX-EUS loaded with highly enriched uranium and low plutonium content. This has to be investigated in order to avoid bias when connecting criteria with input variables. Indeed, the value of  $x_E$  for those simulations is not correct since some PWR MOX-EUS should be counted as PWR UOX. To avoid these simulations, a cut has been set on the plutonium fraction in fresh MOX-EUS fuel

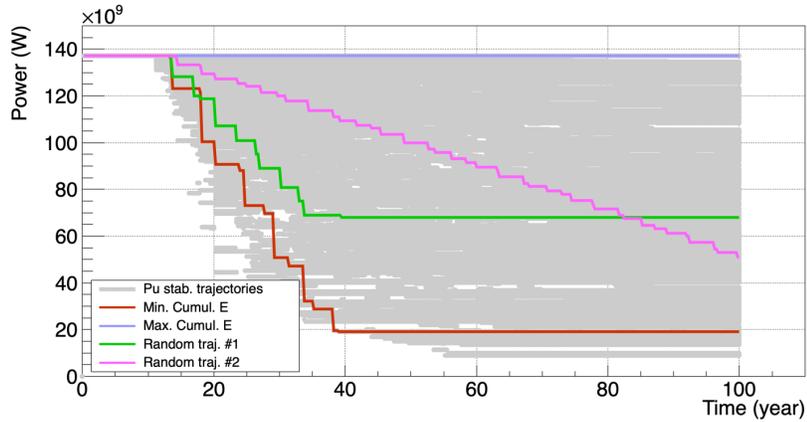


Figure 11: Thermal power evolution for the FiFo data set obtained from the application of plutonium stabilization criteria. Gray lines represent the whole data set. The red line (respectively blue line) is the trajectory that involves the smallest (respectively highest) value of  $\Delta E_{th}$ . Two random examples (green and pink lines) have been added.

435  $\langle \text{Pu Frac.} \rangle < 7\%$ . Furthermore, some plutonium stabilizing simulations correspond to the trivial solution of short term nuclear phase out. A cut on cumulated thermal energy has also been set ( $\Delta E_{th} > 20 \cdot 10^3 \text{TWh}$ ).

The remaining simulations present an effective stabilization of plutonium inventory. The most important variable to achieve this goal is the PWR MOX-EUS fraction in the fleet,  $x_E$ . Plutonium stabilization criteria lead to a normal distribution for  $x_E$  centered in the value 30%. The interaction with other variables is not negligible and has to be taken into account. The nuclear power evolution does not affect the feasibility of the plutonium stabilization. This goal is achievable whatever the future of nuclear power is. Nevertheless, this induces the production of MA, between 40% and 100% of the plutonium mass at EoS, so after 80 years of MOX-EUS utilization.

445

## 6. Transuranic stabilization

In this section, the simultaneous stabilization of the plutonium and MA will be investigated. There is no MA reprocessing in the scenarios considered in this work (see Figure 3). As a consequence, MA net production rate evolution is always positive

450 even for phase-out trajectories since  $^{241}\text{Pu}$  decreased to  $^{241}\text{Am}$ . The stabilization of  
transuranic elements involves then a reduction of plutonium inventory that compensates  
exactly the MA production.

### 6.1. Stabilization criteria

Transuranic stabilization criteria are based on the output observable  $\Delta M_{\text{Pu+MA}}^{\text{tot}}(\text{A})$ ,  
455  $\Delta M_{\text{Pu+MA}}^{\text{tot}}(\text{B})$  and  $\Delta M_{\text{Pu+MA}}^{\text{tot}}(\text{C})$ . In order to have an appropriate number of valid tra-  
jectories, stabilization criteria have been defined as similarly to plutonium stabilization  
criteria:

$$-20 \text{ tons} < \Delta M_{\text{Pu+MA}}^{\text{tot}}(\text{A, B, C}) < 20 \text{ tons} \quad (4)$$

This means that the sum of plutonium and MA mass deviation is constrained between  
-20 and +20 tons. The parallel plot composed by variables of interest is represented  
460 on Figure 12. Three cuts on the variable  $\langle \text{Pu Frac.} \rangle$  have been imposed in order to  
highlight the PWR MOX-EUS characteristics.

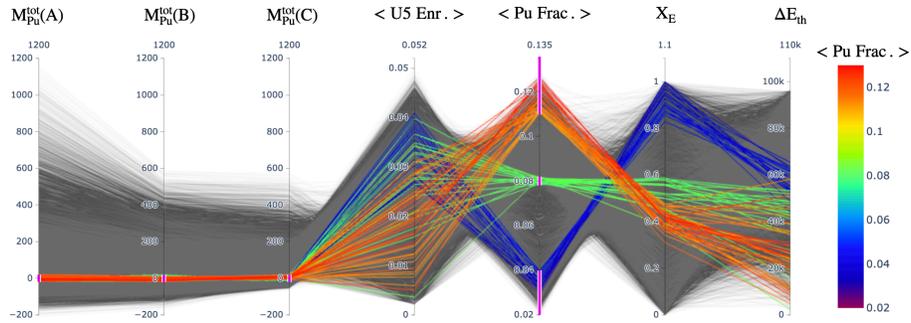


Figure 12: Parallel plot built from the FiFo data set. Gray lines represent all the simulations. Colored lines are trajectories that stabilize the total plutonium mass. Colored right scale is a color code based on the plutonium fraction at beginning of the reactor cycle averaged on each trajectory. Each vertical line is a variable defined in the text. For readability, three cuts have been imposed on  $\langle \text{Pu Frac.} \rangle$ .

As for the case of plutonium stabilization, those three cuts show that some very low  $\langle \text{Pu Frac.} \rangle$  trajectories, representative of PWR UOX rather than MOX-EUS, are associated with very high value of  $x_E$  and a significative value of  $\Delta E_{\text{th}}$ . This suggests that

465 some trajectories seem to stabilize transuranic elements with a high fraction of MOX-EUS and a high energy produced. Nevertheless, those trajectories are characterized by a bias on the  $x_E$  value and have to be removed from the analysis. For this purpose, the < Pu Frac. > variable is imposed to be higher than 7%. Some trajectories, represented by a very small value of  $\Delta E_{th}$ , are solutions of the stabilization conditions. Those  
470 trajectories are closed to fast nuclear phase out and are as a consequence trivial solutions. For this reason, trajectories with  $\Delta E_{th}$  smaller than 20000TWh are also removed from the analysis.

## 6.2. Input variables distribution

After applying criteria described above, the final distributions for plutonium and  
475 MA stabilization are shown on Figure 13. The most important input variable is  $x_E$ . For this reason, input variables are represented in interaction with it.  $x_E$  distribution (top left plot) shows that around 50% of PWR MOX-EUS are required to stabilize the total transuranic inventory. The interaction between  $x_E$  and  $x_P$  (top right plot) is small, slightly positive, and  $x_P$  ranges between 0 and 50%. The interaction with  $E_D$  is also  
480 small but shows a positive relation with  $x_E$ . The representation of  $\Delta E_{th}$  versus  $x_E$  shows a positive relation. Indeed, a high reduction of the nuclear power during the scenario involves a smaller fraction of PWR MOX-EUS, around 40%, while around 60% are required in the high-energy production scenarios that stabilize plutonium and minor actinides. As for the case of the plutonium inventory stabilization, this is mainly due  
485 to a low plutonium availability, in high energy producer trajectories, which leads a low plutonium fraction in MOX-EUS fresh fuel at the reactor BoC. As a consequence, a higher fraction of the power has to be dedicated to PWR MOX-EUS to compensate plutonium production in PWR UOX.

Finally, the interaction between the BU of the PWR UOX and  $x_E$  is small and  
490 slightly negative. Again, a smaller BU produces more plutonium and with a high fissile content. This means more PWR MOX-EUS for the transuranic stabilization.

The input variables characteristics are computed on the Table 5. If the PWR MOX-EUS fraction has to be around 50%, the final thermal power ( $x_P$ ) average is 21% of the initial thermal power with a final end of power decrease average time ( $E_D$ ) close to 60

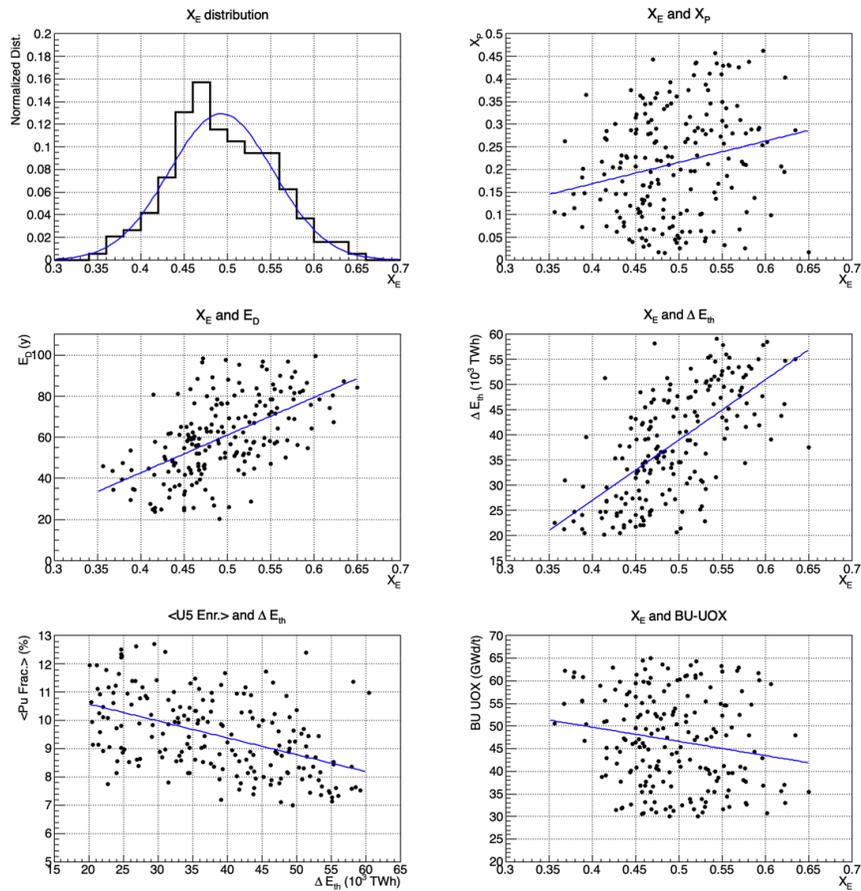


Figure 13:  $x_E$  final distribution for the FiFo data set induced by the transuranic (plutonium and minor actinides) stabilization (top left) fitted by a normal distribution. Interaction of  $x_E$  with  $x_p$  (top right),  $E_D$  (middle left),  $\Delta E_{th}$  (middle right) and BU UOX (bottom right) are represented. The bottom left plot represents the average fraction of plutonium,  $\langle \text{Pu Frac} \rangle$  versus  $\Delta E_{th}$ . First-order polynomial fitted functions are also represented in blue.

495 years. This tends to show that the decrease of the power is required to get trajectories that stabilize transuranic elements. Indeed, if the power decreases, the mass of the PWR MOX-EUS is also decreasing. Thus, a smaller plutonium mass is required in the fresh fuel. This is compatible with the decrease of plutonium inventory. In this way, the plutonium incineration is maximized.

500 Criteria applied to characterize transuranic stabilization can also be used to represent

Table 5: Statistical parameters of input variable distributions for plutonium and minor actinides stabilization obtained from the FiFo data set.

Variable	$x_E$	$x_P$	$E_D$	BU UOX	$\Delta E_{th}$
mean	$4.9 \cdot 10^{-1}$	$2.1 \cdot 10^{-1}$	$6.0 \cdot 10^1$	$4.7 \cdot 10^1$	$3.8 \cdot 10^4$
std	$5.8 \cdot 10^{-2}$	$1.2 \cdot 10^{-1}$	$1.9 \cdot 10^1$	$1.0 \cdot 10^1$	$1.1 \cdot 10^4$
min	$3.6 \cdot 10^{-1}$	$1.5 \cdot 10^{-2}$	$2.0 \cdot 10^1$	$3.0 \cdot 10^1$	$2.0 \cdot 10^4$
max	$6.5 \cdot 10^{-1}$	$4.6 \cdot 10^{-1}$	$9.9 \cdot 10^1$	$6.5 \cdot 10^1$	$6.0 \cdot 10^4$

transuranic elements evolution on the scenario. The figure 14 shows transuranic mass evolution and contributions for plutonium and MA. It is clearly shown that all trajectories are plutonium burner in order to compensate MA production. At the end of the scenario after 100 years of operation, the MA mass is very close to plutonium mass; around 300

505 tons.

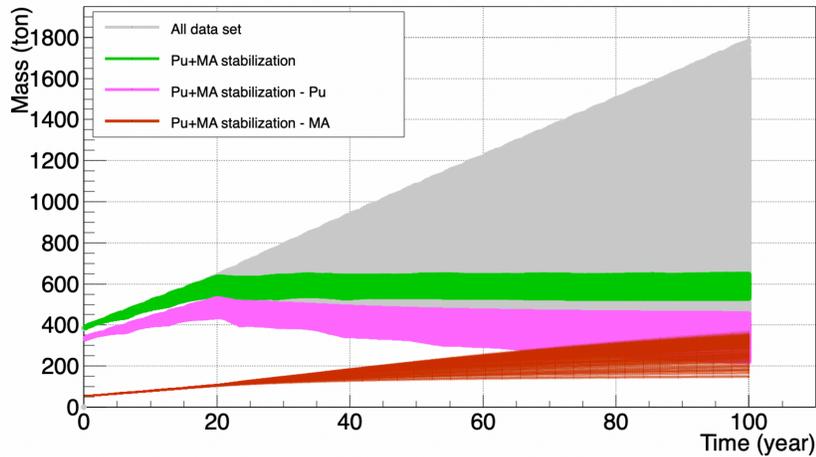


Figure 14: Total transuranic mass evolution in the fleet (gray lines). The green lines represent the sum of plutonium and minor actinides masses for trajectories obtained by stabilization criteria. Purple lines represent plutonium mass evolution and red lines represent minor actinides mass evolution after the application of the same criteria.

The figure 15 shows trajectories of the data set obtained after transuranic stabilization

criteria applications.

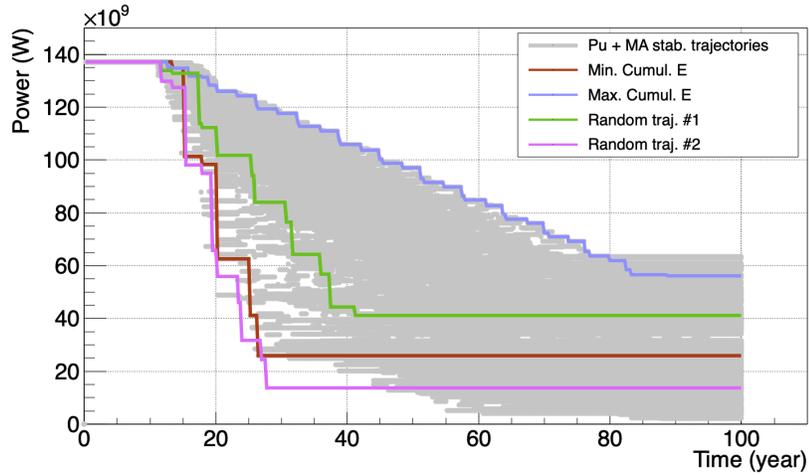


Figure 15: Thermal power evolution for the FiFo data set obtained from the application of plutonium stabilization criteria. Gray lines represent the whole data set. The red line (resp. blue line) is the trajectory that involves the smallest (resp. highest) value of  $\Delta E_{th}$ . Two random examples (green and pink lines) have been added.

In agreement with results obtained before, it is shown that the stabilization of transuranic elements are associated to the decrease of the power. This observation  
 510 can be explained by the fact that transuranic stabilization involves plutonium consumption. As a consequence, the plutonium availability decreases and the reduction of the power follows. When the fleet is reaching a small thermal power, transuranic elements production and incineration are very small and the sum of plutonium and MA is stable.

### 6.3. Transuranic stabilization summary

515 Similarly to the plutonium stabilization related section, a condition has been imposed on the plutonium content of PWR MOX-EUS in order to avoid important bias of the input variable  $x_E$ . Application of transuranic stabilization criteria shows that around 50% of PWR MOX-EUS are required. This value depends on the power evolution of the nuclear fleet. The stabilization is composed of an increase of MA inventory  
 520 compensated by a decrease of plutonium inventory. A decrease of the nuclear power is necessary in order to fit with plutonium inventory decrease.

Also, increasing the consumption rate of the plutonium is possible and leads to the decrease of the transuranic inventory. Nevertheless, this involves a higher fraction of PWR MOX-EUS in the fleet and a fast decrease of the nuclear power. The reader can refer to [27] for further details on transuranic decrease feasibility.

## 7. Conclusions

This work aims to study the potential of the plutonium multi-reprocessing in PWRs, in terms of inventory management. The French nuclear fleet has been considered and simulated in order to compose the initial condition of simulation design of experiments.

A context of deep uncertainty characterizes the future of the French electro-nuclear fleet, since the decision on the construction of an SFR demonstrator has been postponed to the end of the 21st century. In this framework, several options could be examined. If a short-term phase out was considered, there would have no incentive for developing highly innovative technology for plutonium multi-recycling. However, if nuclear energy is planned beyond the medium term and no SFR construction is considered, the plutonium and MA inventory should be controlled, at worst stabilized, at best minimized in the current technology. This is the basic assumption of this work that drives the nuclear fleet evolution possibilities.

The multi-reprocessing of plutonium in PWR is a technical challenge. The plutonium quality decreases with irradiation and safety margins may be strongly affected. As a consequence, innovative assembly designs are required to achieve this goal. In this work, the MOX fuel on enriched uranium support is considered. A design of experiments characterized by wide ranges for reactors technical parameters has been used. This methodology aims to be representative of several possible futures. A high amount of fuel cycle simulations has been performed with the fuel cycle simulator CLASS. A preliminary analysis based on sensitivity indices estimation has shown that the most important input variable that drives the plutonium production is the PWR MOX-EUS fraction in the fleet. The nuclear power evolution is also important. Interactions between input variables are slightly significative.

The analyze of the set of simulations allows highlighting technical conditions re-

quired to stabilize plutonium and transuranic (plutonium and minor actinides) elements.

We have shown that around 30% of PWR MOX-EUS are required to stabilize the plutonium inventory on a 100-year scenario. This goal is achievable whatever the nuclear power evolution is. For those scenarios, the multi-reprocessing of plutonium  
555 involves a high production of minor actinides.

The stabilization of transuranic elements conditions leads to an averaged fraction of PWR MOX-EUS close to 50%, significantly higher than the case of plutonium stabilization. The difference comes from the nuclear power evolution. The transuranic stabilization imposes a decrease of the plutonium inventory to compensate the minor  
560 actinides production. As a consequence, a decrease of the total nuclear power is required. The decrease fits with the plutonium availability. Trajectories range from a fast phase out up to a fleet with half of the initial power. This result is obtained on a 100-year scenario and cannot be generalized beyond 100 years.

In this work, simulation simplifications have been done and would require more  
565 discussions. For instance, only macro-reactors are considered. In this approach, each reactor is combined and gathered into a unique reactor by type of fuel in the problem. It is shown in [27] that results obtained from the macro-reactors approach are conserved in a full-size fleet with a high number of standard reactors. Another example, the design of experiment is based on an instantaneous MOX-EUS fueling at the park scale, which  
570 is not realistic. Here again, a more complex simulation including several reactors and a progressive MOX-EUS deployment provides similar results [27].

More generally speaking, the impact of the simulation simplifications on results should be precisely investigated. Some international effort, such as the FIT (Functionality Isolation Test) project [36] aims to quantify biases produced by simulation  
575 simplifications.

Forthcoming works could go further in safety studies related to multi-reprocessing of plutonium in PWR. In the present work, safety issues related to high plutonium content in the MOX-EUS fuel are taken into account by setting the maximum plutonium content to a value sampled between 8% and 13%. It has been demonstrated that the conclusions  
580 of this study are non-impacted by this parameter. This approach could be developed in future works by connecting the isotopic composition of the fuel with an estimated

value of the maximum plutonium content calculated from a full-core simulation. In addition, an essential and required step would be to examine the impact of the plutonium multi-reprocessing on the fuel cycle facilities.

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