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# EQUILIBRIUM RECONSTRUCTION FROM DISCRETE MAGNETIC MEASUREMENTS IN A TOKAMAK

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## ABSTRACT

We describe an algorithm for the reconstruction of the equilibrium in a Tokamak from discrete magnetic measurements. In order to solve this inverse problem we first use toroidal harmonics to compute Cauchy boundary conditions on a fixed closed contour. Then we use these Cauchy boundary conditions to solve a non-linear source identification problem.

## 1. INTRODUCTION

In this paper we are interested in the numerical reconstruction of the quasi-static equilibrium of a plasma in a Tokamak [1]. The state variable of interest in the modelization of such an equilibrium under the axisymmetric assumption is the poloidal flux  $\psi(r, z)$  which is related to the poloidal magnetic field by the relation  $B = \frac{1}{r} \nabla \psi^\perp$  in the cylindrical coordinate system  $(r, z)$ .

A poloidal cross section of a Tokamak and the different domains and contours are shown on Fig. 1. The domain  $\Omega_0$  contains the poloidal field coils (PFcoils) domains  $\Omega_{C_i}$  and the plasma domain  $\Omega_p$ . It is assumed that  $\Omega_0$  does not include any ferromagnetic structure and thus the poloidal flux satisfies the elliptic PDE

$$-\Delta^* \psi = j \quad (1)$$

where the differential operator

$$\Delta^* = \frac{\partial}{\partial r} \left( \frac{1}{\mu_0 r} \frac{\partial}{\partial r} \right) + \frac{\partial}{\partial z} \left( \frac{1}{\mu_0 r} \frac{\partial}{\partial z} \right),$$

is linear and  $j$  is the toroidal component of the local current density.

In  $\Omega_0 \setminus \{\Omega_p \cup \Omega_{C_i}\}$  the current is null ( $j = 0$ ), in the PFcoils  $\Omega_{C_i}$  it is supposed to be known ( $j = I_i/S_i$ ,  $S_i$  is the surface of coil  $\Omega_{C_i}$  and  $I_i$  the total intensity

of the current) and in the plasma  $\Omega_p$  it is unknown but takes the form

$$j = rp'(\psi) + \frac{1}{\mu_0 r} (ff')(\psi) \quad (2)$$

In  $\Omega_p$  Eq. (1) is called the Grad-Shafranov equation and  $p'$  and  $ff'$  are unknown functions to be identified.

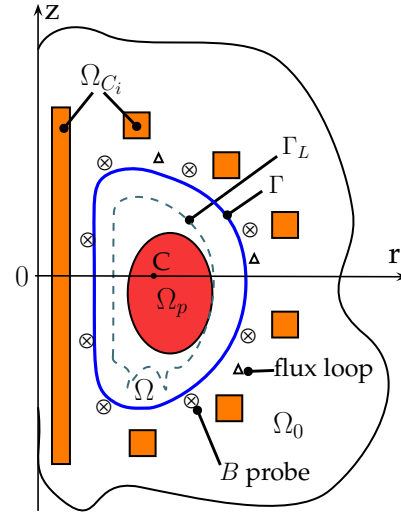


Figure 1: Poloidal cross section of a Tokamak.

The plasma domain is unknown,  $\Omega_p = \Omega_p(\psi)$ . It is a free boundary problem in which the plasma boundary is defined either by its contact with the limiter  $\Gamma_L$  (as in Fig. 1) or as a magnetic separatrix (hyperbolic line with an X-point).

In order to achieve the numerical reconstruction of the equilibrium the main inputs we have are magnetic measurements taken at several locations surrounding the vacuum vessel (see Fig. 1): B probes measure the

local value of the poloidal magnetic field and flux loops measure the local value of the flux  $\psi$ .

The method we propose can be divided into two main steps described in the next section.

## 2. NUMERICAL METHOD

In a first step we solve Eq. (1) in  $\Omega_0 \setminus \Omega_p$  using an analytic solution and a fit to the magnetic measurements. The goal is to transform the set of discrete measurements into Cauchy conditions for  $\psi$  on a fixed contour  $\Gamma$ . This latter defines a domain  $\Omega$  of boundary  $\Gamma$  and containing the plasma (see Fig. 1). In a second step the value of  $\psi$  on  $\Gamma$  is used as a Dirichlet boundary condition to solve numerically Eq. (1) in  $\Omega$ , whereas the remaining Neumann boundary condition on  $\Gamma$  is used simultaneously for the identification of the unknown functions  $p'$  and  $ff'$ .

### 2.1. Step 1 - Compute Cauchy conditions on $\Gamma$

Each of the PFcoils is modeled by a sum of filaments of current [3]. Using the analytic expression of the Green function of the operator  $\Delta^*$  and the additivity property we can subtract from  $\psi$  and from the magnetic measurements the effects of the PFcoils.

In the domain  $\Omega_0 \setminus \Omega_p$  the resulting corrected flux then satisfies  $-\Delta^*\psi = 0$ . Using a system of toroidal coordinates with center C inside the plasma domain and a separation of variable technique, any solution of this equation can be shown to be equal to a series of toroidal harmonic functions  $T_n$  [5, 6]. Numerically these harmonics can be accurately evaluated [4] and the flux can be efficiently approximated using a truncated series,  $\psi = \sum_{n=1}^N a_n T_n$ . The coefficients  $a_n$  are computed by a least-square fit to the modified magnetic measurements. A regularization term can be added to the least-square cost function for example imposing some regularity on a circular contour surrounding the center of the toroidal coordinates system.

Finally one can evaluate  $(\psi, \partial_n \psi)$  on the contour  $\Gamma$  and thus provide Cauchy conditions  $(g, h)$  to the resolution of the problem in the domain  $\Omega$ .

### 2.2. Step 2 - Reconstruction in $\Omega$

The Dirichlet boundary condition  $g$  is used to solve the boundary value problem:

$$\begin{cases} -\Delta^*\psi &= \lambda \left[ \frac{r}{R_0} A(\bar{\psi}) + \frac{R_0}{r} B(\bar{\psi}) \right] \chi_{\Omega_p(\psi)} & \text{in } \Omega \\ \psi &= g & \text{on } \Gamma \end{cases} \quad (3)$$

where the unknown functions  $A$  and  $B$  are related to  $p'$  and  $ff'$ ,  $\bar{\psi}$  is a normalized flux, and  $\lambda$  and  $R_0$  are normalizing coefficients. The Neumann boundary condition is used to identify  $A$  and  $B$  by minimizing the cost function  $J(A, B) = \int_{\Gamma} (\partial_n \psi - h)^2 ds + R$  where  $R$  is a Tikhonov regularization term.

An iterative strategy involving a finite element method for the resolution of the direct problem (3) and an optimisation procedure for the identification of the non-linearity is proposed [2]. It is important to achieve this identification within a few ms so as to be able in the future to control the current profile in real time. The main ideas are: pre-computation of the inverse of the finite element stiffness matrix and of all the elements that are not modified by the non-linearities, Picard iterations for these non-linearities, reduction of the functions to be identified in small dimension basis and least-square resolution by normal equations.

## 3. CONCLUSION

The method presented here has led to the development of a software, EQUINOX, which enables to follow in real-time the quasi-static evolution of the plasma equilibrium in any Tokamak. It has already been validated on TORE SUPRA (the CEA-EURATOM Tokamak at Cadarache), JET (Joint European Torus) or ITER configurations.

## 4. REFERENCES

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