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GADOLINIUM-LOADED PLASTIC SCINTILLATORS FOR THERMAL NEUTRON DETECTION AND COUNTING USING COMPENSATION

J. Dumazert^{1*}, G. H. V. Bertrand¹, R. Coulon¹, M. Hamel¹, V. Kondrasovs¹, F. Sguerra¹, C. Dehé-Pittance¹, S. Normand¹, L. Méchin².

¹CEA, LIST, Laboratoire Capteurs Architectures Electroniques, 91191 Gif-sur-Yvette, France.

²CNRS, UCBN, Groupe de Recherche en Informatique, Image, Automatique et Instrumentation de Caen, 14050 Caen, France.

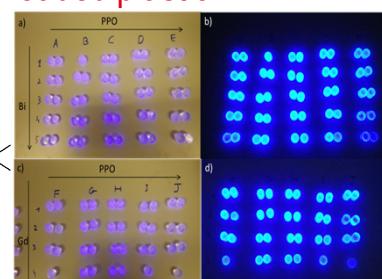
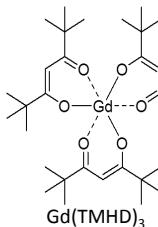
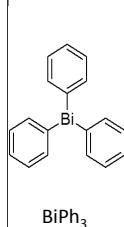
Introduction



Helium 3 neutron counter

Neutron counting forms a critical branch of **nuclear-related issues**, whether dose rate monitoring for radioprotection or radiological material detection addressing **CBRN threats** are concerned. More specifically, the last decade has been driven by the quest for competitive **alternative technologies to neutron counters based on the helium 3 isotope**, whose worldwide shortage has generated massive market value fluctuations. The loading of plastic scintillators with thermal neutron absorbing elements, such as **gadolinium**, represents a **cost-effective and scalable strategy**.

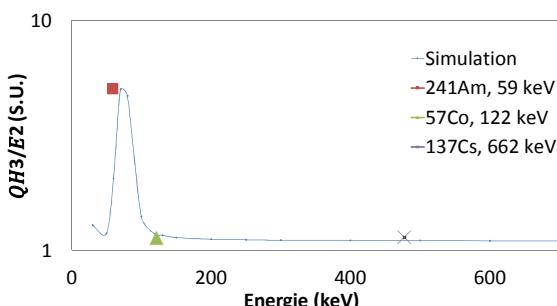
Gadolinium and bismuth loaded plastic scintillators



- Small **equal-volume** and **same-geometry** gadolinium ($\text{Gd}(\text{TMHD})_3$ compound) and bismuth (BiPh_3 compound) loaded scintillators (Left figure) [1]
- Diameter $d = 17.5$ mm and height $h = 8.1$ mm
- H3 sample: Density $\rho = 1.1 \text{ g.cm}^{-3}$, **2 %wt gadolinium**
- E2 sample: Density $\rho = 1.0 \text{ g.cm}^{-3}$; **1 %wt bismuth**

Right figure : a) and b) Bismuth-loaded plastic scintillators with various loadings in BiPh_3 and PPO under UV ; c) and d) Gadolinium-loaded plastic scintillators with various loadings in $\text{Gd}(\text{TMHD})_3$ and PPO under UV illumination.

Counting method using gamma compensation

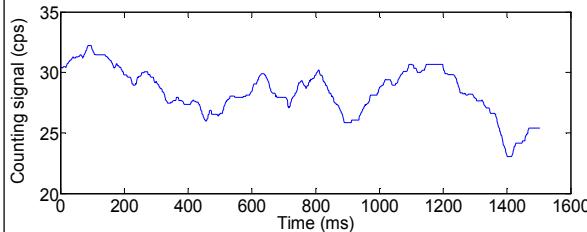


- Thermal neutron radiative capture by **157** and **155 gadolinium isotopes** :

$$^{157}_{64}\text{Gd} + ^1\text{n} \rightarrow ^{158}_{64}\text{Gd}^* \rightarrow ^{158}_{64}\text{Gd} + \gamma + \text{XR} + \text{ICe}^- + \text{Ae}^-$$
 (255000 b)

$$^{155}_{64}\text{Gd} + ^1\text{n} \rightarrow ^{156}_{64}\text{Gd}^* \rightarrow ^{156}_{64}\text{Gd} + \gamma + \text{XR} + \text{ICe}^- + \text{Ae}^-$$
 (61000 b)
- Prompt gamma rays and IC electrons** emitted in (n,γ) radiative cascade, notably in $F_1 = [0 ; 100 \text{ keV}]$ and $F_2 = [100 \text{ keV} ; 200 \text{ keV}]$ energy ranges
- H3 **gamma and neutron** sensitive, E2 **solely gamma** sensitive : **compensation** of the gamma response of H3 by the response of E2 [2]
- The gamma compensation **coefficient** $Q = \frac{N_{H3}}{N_{E2}}$ is a function of the incident **gamma ray energy** E_γ
- Agreement** between experimentally obtained values for Q and MCNPX Monte-Carlo code simulations

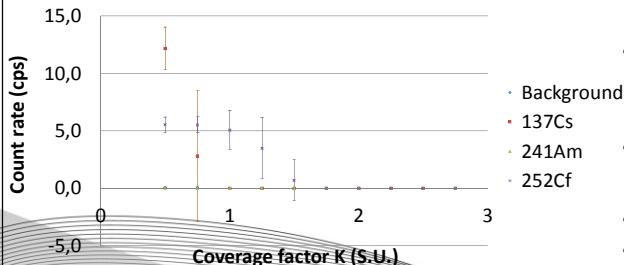
Nonlinear smoothing and hypothesis test



- Digitization every $\Delta t = 100$ ms over $T = 180$ s, nonlinear **Centered Skellam Test smoother** [3] for Poisson variance reduction: $\widehat{N_{H3}}$ and $\widehat{N_{E2}}$
- Hypothesis test for neutron counting parameterized by a **coverage factor** K :

$$\text{If } \widehat{N_{H3}} - \bar{Q} \cdot \widehat{N_{E2}} > K \cdot \sqrt{\sigma^2(\widehat{N_{H3}}) + \bar{Q}^2 \cdot \sigma^2(\widehat{N_{E2}}) + \sigma_{E_\gamma}^2(\bar{Q}) \cdot \widehat{N_{H3}}^2}$$
 Then $\widehat{N_n} = \widehat{N_{H3}} - \bar{Q} \cdot \widehat{N_{E2}}$
 Else $\widehat{N_n} = 0$ (every Δt)
- Moving median** over $\Delta T = 30$ s for variance reduction on $\widehat{N_n}$

Results and conclusions



- Essentially **high energy gamma background** of ^{252}Cf source **compensated** over F_1 and F_2 for $K = 1$
- Count rate $\widehat{\lambda_n} = 13.6 \text{ cps over } F_1 \text{ for } K = 1.25$, guarantees null count rate for the room and ^{137}Cs backgrounds, but falsely counts for ^{241}Am background
- Count rate $\widehat{\lambda_n} = 5.1 \text{ cps over } F_2 \text{ for } K = 1$, guarantees null count rate for the room, ^{137}Cs and ^{241}Am background
- Compares to $\widehat{\lambda_n} = 42.3 \text{ cps}$ helium 3 count rate (65NH45 model)
- Scale-up** for higher precision and **alternative to bismuth compensation**