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# Microstructural changes induced by low energy heavy ion irradiation in titanium silicon carbide

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## ABSTRACT

Low energy ion irradiation was used to investigate the microstructural modifications induced in  $\text{Ti}_3\text{SiC}_2$  by nuclear collisions. Characterization of the microstructure of the pristine sample by electron back-scatter diffraction (EBSD) shows a strong texturing of  $\text{TiSi}_2$ , which is a common secondary phase present in  $\text{Ti}_3\text{SiC}_2$ . A methodology based on atomic force microscopy (AFM) was developed to measure the volume swelling induced by ion irradiation, and it was validated on irradiated silicon carbide. The swelling of  $\text{Ti}_3\text{SiC}_2$  was estimated to  $2.2 \pm 0.8$  % for an irradiation dose of 4.3 dpa at room temperature. Results obtained by both EBSD and AFM analyzes showed that nuclear collisions induce an anisotropic swelling in  $\text{Ti}_3\text{SiC}_2$ .

Keywords:  $\text{Ti}_3\text{SiC}_2$ , nuclear interaction, electron back-scatter diffraction, atomic force microscopy, anisotropic swelling

PACS: 81.05.Je, 61.80.Lj, 25.75.Ag, 28.41.Bm, 68.37.Ps, 61.05.J-

## 1. Introduction

The Gas-cooled Fast Reactor (GFR) is one of six new systems studied in the framework of the Generation IV International Forum (GIF). These systems are characterized by an increased security level, a better economic competitiveness, and the ability to recycle all the fuel in order to upgrade it to a fissionable material and to minimize long-lived waste production by transmutation [1]. The GFR is designed to work under helium-pressure and at high-temperature (1100-1300 K). Due to these working conditions, non-oxide refractory ceramics are required as fuel cladding. Thus, carbides turn out to be excellent candidates due to their remarkable mechanical and thermal properties. However, their behavior under irradiation has to be investigated.

Among potential carbides, ternary  $\text{Ti}_3\text{SiC}_2$  presents some interesting properties. In 1972 Nickl *et al.* [2] remarked that this material is abnormally soft for a carbide, so that its hardness decreases as the applied load increases. For this reason Goto *et al.* [3] qualified  $\text{Ti}_3\text{SiC}_2$  as a “ductile ceramic”. Furthermore,  $\text{Ti}_3\text{SiC}_2$  combines the properties of metals with those usually attributed to ceramics [4-7]. Thus, this material is not only soft but also stiff and tough, it behaves as a good electrical and thermal conductor, and it can be easily machined with the tools generally used for steels.

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The interesting mechanical properties of  $\text{Ti}_3\text{SiC}_2$  suggest that this compound could be used as fuel cladding material. Its damage tolerance to mechanical stresses might indicate a high resistance to irradiation. Nevertheless, apart from few recently published articles related to  $\text{Ti}_3\text{SiC}_2$  [8-10], and  $\text{Ti}_3(\text{Si},\text{Al})\text{C}_2$  [11-14], few information is available about its behavior under irradiation.

Previously [15,16], we showed that an irradiation performed at room temperature with 4 MeV Au ions to a fluence of  $10^{19} \text{ m}^{-2}$  induces both an erosion of the  $\text{Ti}_3\text{SiC}_2$  grain boundaries, as observed by scanning electron microscopy, and a revealing of the grain structure, as evidenced by atomic force microscopy. We attributed the former phenomenon to a preferential sputtering due to lower threshold displacement energy of the atoms located in grain boundaries. For the latter result, we were led to consider the occurrence of preferential sputtering as a function of the crystallite orientation. In this work, complementary irradiation experiments suggest another explanation.

## 2. Experimental

The polycrystalline samples were provided by the 3-ONE-2 company (Vorhees, NJ, USA). They consist of about 74 %  $\text{Ti}_3\text{SiC}_2$ , 19 %  $\text{TiC}_{0.92}$ , and 7 %  $\text{TiSi}_2$  (as estimated by X-ray diffraction). As-received samples were polished with diamond paste of a size down to 1 micron.

The interactions occurring in reactors are essentially elastic (or nuclear) collisions due to primary knock-on atoms from neutrons, and recoil atoms arising from alpha-decays. In order to simulate these interactions, low energy ion irradiations are usually performed. Thus, the polished face of the samples was irradiated with 4 MeV Au ions provided by the ARAMIS accelerator (CSNSM-Orsay, France). Table 1 summarizes the irradiation conditions.

Table 1: Irradiation conditions.

Temperature	Room Temperature	773 K	1223 K
Fluence ( $\text{m}^{-2}$ )	$10^{16}, 10^{17}, 10^{18}, 10^{19}$	$10^{16}, 10^{17}, 10^{18}, 10^{19}$	$10^{19}$

In order to compare the results of these irradiations with previous data (especially those using neutron irradiations), it is usual to deal with the number of displacements per atoms (dpa) of the target, induced by the irradiation as a function of the depth within the irradiated material. The fluence scale has been converted into a dpa scale on the basis of TRIM-2008 calculations [17], by considering the number of vacancies produced with 4 MeV Au ions as a function of depth in  $\text{Ti}_3\text{SiC}_2$ . The displacement energies were: 25 eV for Ti, 15 eV for Si, and 28 eV for C. Figure 1 shows the variation of the damage level (in dpa per fluence unit) and of the ion distribution (also estimated with the TRIM code) as a function of the depth in the target material. This Figure shows that the damaged thickness may be estimated to 760 nm. In this layer, irradiation induces an average dpa per fluence unit of  $4.3 \times 10^{-19} \text{ m}^2$ , viz. 4.3 dpa for an irradiation to  $10^{19} \text{ m}^{-2}$ .

Different techniques were used to characterize the surface modifications induced by ion irradiation. Atomic force microscopy (AFM) aims both to analyse the surface topography modifications and to measure the swelling. Field emission gun scanning electron microscopy (FEG-SEM) was used to underline differences between  $\text{Ti}_3\text{SiC}_2$  and the other phases by imaging the surface of samples with back-scattered electrons. Coupled to FEG-SEM, electron back-scatter diffraction (EBSD) was used to characterize the crystallites before irradiation. EBSD is a powerful technique for the quantification of both the microtexture and the microstructure of polyphased crystalline materials.

For EBSD, as-received samples were also polished with diamond suspensions down to 1  $\mu\text{m}$ . Then, they were polished with  $\frac{1}{4} \mu\text{m}$  colloidal silica suspension for 3 hours. EBSD analyzes were carried out using an HKL Technology (now Oxford Instruments) system installed on a Zeiss Supra 55 VP FEG-SEM operating at 17-20 kV and a probe-current of about 20 nA. EBSD analyzes were not possible on irradiated samples because of the loss of crystallinity induced by nuclear collisions [8,15].

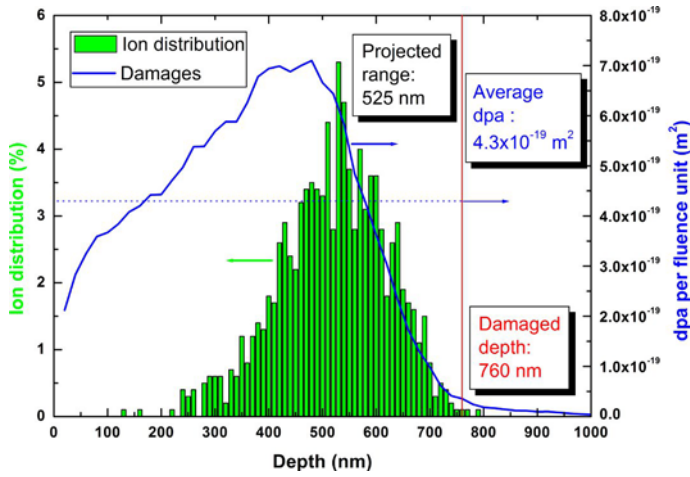


Figure 1: Depth distribution of implanted ions and number of displacements per atom (per fluence unit) for  $\text{Ti}_3\text{SiC}_2$  irradiated with 4 MeV Au ions.

Crystal structure data were created using the Twist add-on with the data shown in Table 2. In this Table,  $\text{Ti}_I$  atoms correspond to the atoms of the basal planes linking the  $\text{CTi}_6$  octahedrons, and  $\text{Ti}_{II}$  atoms to those bordering the silicon basal planes. Representations of the  $\text{Ti}_3\text{SiC}_2$  lattice can be found elsewhere [4-6]. The complexity of the  $\text{Ti}_3\text{SiC}_2$  diffraction patterns leads to the failure of the automatic indexation algorithm for some particular orientations. More precisely, the band recognition process using the standard Hough transform fails when Kikuchi bands are closely spaced and nearly parallel, which is the case of the  $\text{Ti}_3\text{SiC}_2$  diffraction pattern. Nevertheless, the number of non-indexed patterns is relatively small, leading to a reliable microstructure analysis.

Table 2: Wyckoff positions of the atoms for the three phases present in the studied material.

Phase	$\text{Ti}_3\text{SiC}_2$				TiC		$\text{TiSi}_2$	
Space group	$P6_3/mmc(194)$				$Fm-3m(225)$		$Fddd(70)$	
Atoms	$\text{Ti}_I$	$\text{Ti}_{II}$	Si	C	Ti	C	Ti	Si
Wyckoff positions	2a	4f	2b	4f	4a	4b	8a	16e

### 3. Results and discussion

#### 3.1. Characterization of pristine samples

The characterization of as-prepared (or pristine) samples was partly reported previously [15,16]. Briefly, we first noticed a difference in the contrast between the three phases using back-scattered electrons in the FEG-SEM. Then, by AFM we were able to distinguish TiC from the other phases due to its bigger hardness: TiC grain surfaces appear elevated above those of other phases.

In order to confirm that the microstructure revealed after irradiation at room temperature to  $10^{19} \text{ m}^{-2}$  [15,16] depends on the crystallite orientation, EBSD was performed on unirradiated samples. Figure 2 shows the microstructure of pristine samples as revealed by EBSD. Figure 2a corresponds to the diffraction pattern quality quantified by the “Band Contrast”, while Figure 2b presents the phase distribution (with  $\text{Ti}_3\text{SiC}_2$  in blue, TiC in red and  $\text{TiSi}_2$  in yellow). These mappings show distinct morphologies for the different phases:  $\text{Ti}_3\text{SiC}_2$  appears as strongly faceted and elongated crystallites, while TiC exhibits more equiaxed shapes. The minor  $\text{TiSi}_2$  phase appears as small isolated islands, which are preferentially located near other phase grain corners.

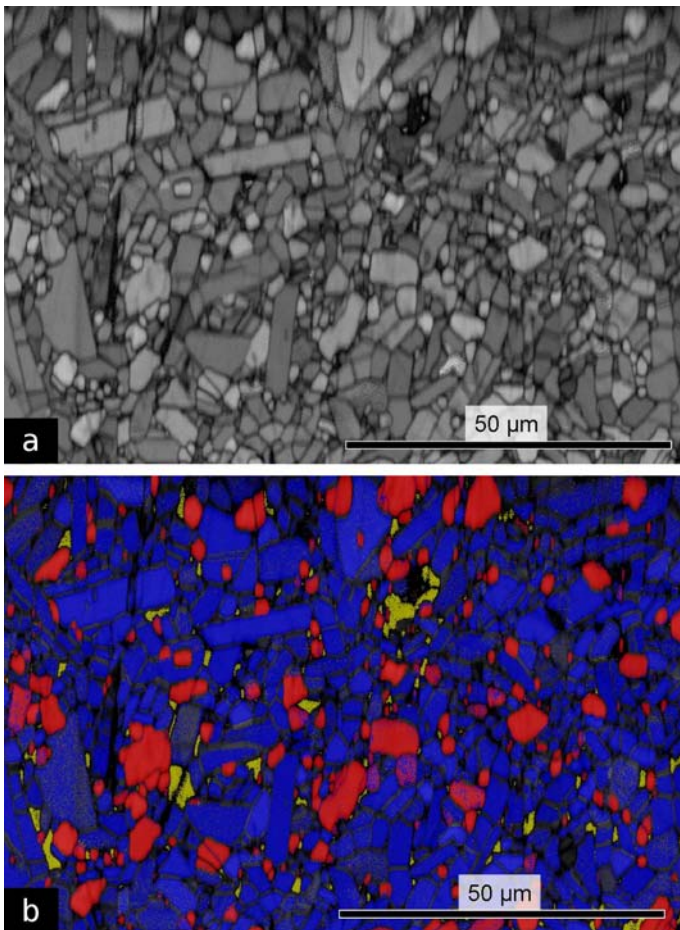


Figure 2: Microstructure of a pristine sample observed by EBSD; (a) micrograph in band contrast, (b) phase distribution:  $\text{Ti}_3\text{SiC}_2$  in blue,  $\text{TiC}$  in red, and  $\text{TiSi}_2$  in yellow.

The crystallographic orientations of the different phases are shown in Figure 3. The color code corresponds to the crystallographic direction parallel to the normal direction as shown in the standard inverse pole figure for each phase. Crystallographic information can be obtained from these mappings. First, by a careful examination of the individual orientations of  $\text{Ti}_3\text{SiC}_2$ , it can be shown that the morphology of the crystallites perfectly matches the hexagonal symmetry of the crystal: the elongated and faceted crystallites (mostly green and blue on Figure 3a) have a c-axis perpendicular to the long straight edges. Other crystallites appearing in orange or red have a c-axis perpendicular to the map section and the faceted edges are perpendicular to the a-axis. From this observation, it can be concluded that this phase forms hexagonal platelets. While the most abundant phases do not have a preferential crystallographic orientation, the minor  $\text{TiSi}_2$  islands appear in the same color: this result indicates that this phase is strongly textured. This is confirmed by the pole figures of the  $\langle 100 \rangle$  directions shown in Figure 4. Other areas of the same sample have been analyzed in order to investigate this particular point. The images confirm that  $\text{TiSi}_2$  is strongly textured at a local scale.

To our knowledge, such a preferential orientation of  $\text{TiSi}_2$  has never been reported in the literature. Since this phenomenon was not observed by low-incidence X-ray diffraction [15], we believe that it is localized. Unfortunately, no explanation has yet been provided to explain such a texturing. However, two reasons may be invoked:

- (i) During the synthesis of  $\text{Ti}_3\text{SiC}_2$  a liquid eutectic Si- $\text{TiSi}_2$  could have formed at 1330 °C [18-20]; Audubert *et al.* [21] have shown that this liquid phase was also formed in the sintered product;
- (ii) The  $\text{TiSi}_2$  phase melts at 1540 °C [22], so that as indicated by Morgiel *et al.* [23] it seems to wet the other grains, filling up the free spaces.

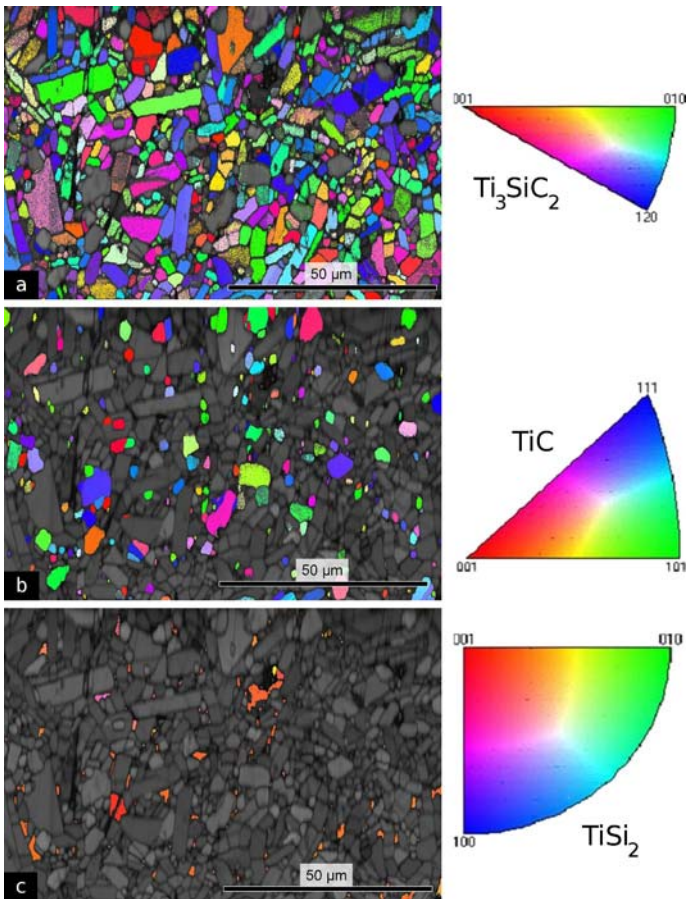


Figure 3: Mapping of the crystallographic orientation of (a)  $\text{Ti}_3\text{SiC}_2$ , (b)  $\text{TiC}$ , and (c)  $\text{TiSi}_2$ .

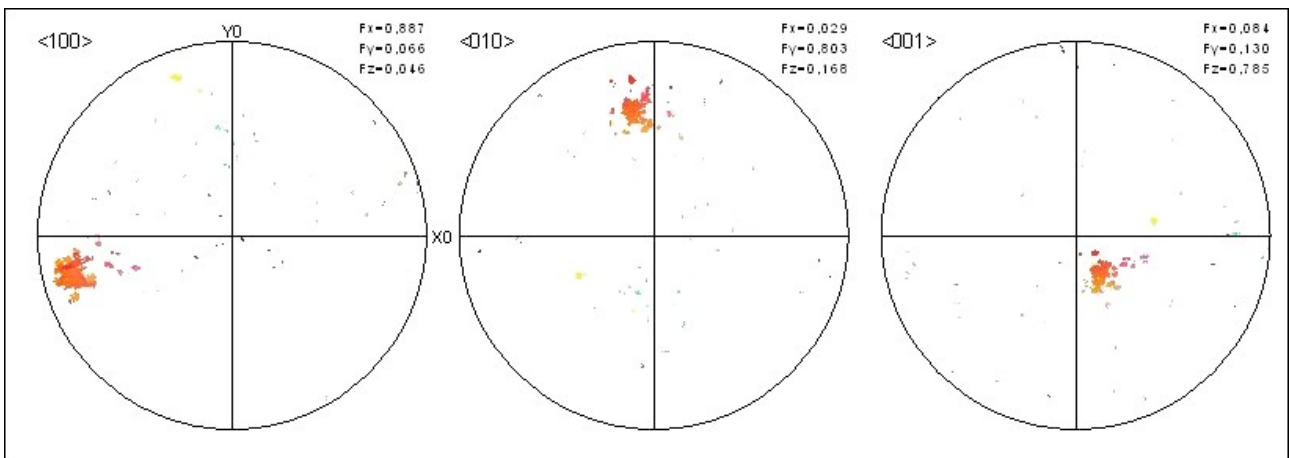


Figure 4:  $\langle 100 \rangle$  pole figures for  $\text{TiSi}_2$  showing the strong texturing of this phase on the map presented in Figure 3c.

No information is available about the sintering temperature of the studied material. However, since the sintering temperature of the  $\text{Ti}_3\text{SiC}_2$  powder generally ranges between 1400 and 1700 °C [7,24], temperatures of 1330 and 1540 °C were probably reached during the fabrication of these samples. This feature could explain the observed preferential orientation by a liquid-solid transition.



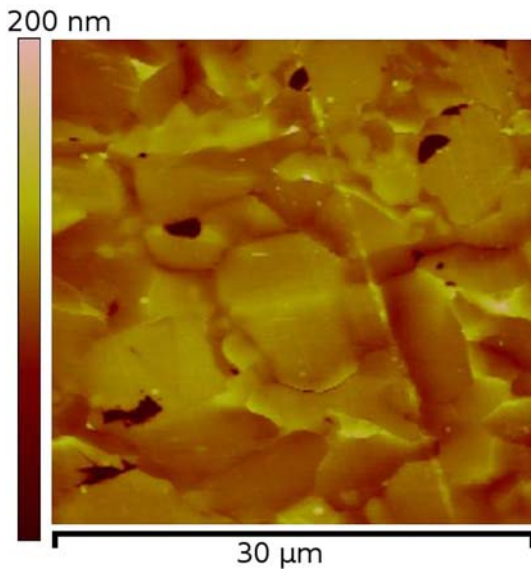


Figure 5: Microstructure revealing observed by AFM in a sample irradiated at room temperature to  $10^{19} \text{ m}^{-2}$ .

Figure 5 shows an example of the microstructure revealing phenomenon previously observed in a sample irradiated at room temperature with 4 MeV Au ions to  $10^{19} \text{ m}^{-2}$  [16]. Characterizing both the shape and the size of the revealed grains by AFM, we showed that these grains are the same as the crystallites that may be observed by EBSD in pristine sample. This confirms that the revealing of the microstructure induced by irradiation is dependent on the crystallite orientation.

### 3.2. Swelling measurements

To measure the swelling induced by irradiation, some samples were partly irradiated by placing a protective aluminum mask on part of these samples to protect them from the ion beam. Figure 6a shows a micrograph, obtained by AFM, of the interface between the virgin and the irradiated areas for an irradiation at room temperature to  $10^{19} \text{ m}^{-2}$ . The lighter shade of the irradiated area suggests that its height is higher than before irradiation, and therefore that irradiation has induced a swelling of the material.

Measuring the sections of partly irradiated grains on several micrographs (see Figure 6b), we estimated an average step of  $16.8 \pm 6.3 \text{ nm}$  between the virgin and irradiated areas for the irradiation at room temperature to  $10^{19} \text{ m}^{-2}$  (the large height difference between the two sections of Figure 6b will be discussed later on). Since the swelling due to ion irradiation induces a change of the sample dimensions only along the ion beam direction [25-27], the measured linear swelling provides an estimation of the volume swelling. To evaluate the linear swelling, other authors compare the step induced by the irradiation either with the projected range [28] or with the damaged thickness [29] (see Figure 1). For a fluence of  $10^{19} \text{ m}^{-2}$ , the highest fluence used in this work, the implanted-ion concentration is not large enough ( $\sim 160 \text{ ppm}$ ) to induce a significant swelling [30]. Since for lower fluences the concentration of implanted ions is less important, it appears that the projected range is not a good parameter to evaluate the volume swelling in this study. However, the observations of cross sections by transmission electron microscopy and the evaluation of lattice parameters by X-ray diffraction have highlighted that nuclear collisions induce both the formation of defects in  $\text{Ti}_3\text{SiC}_2$  [8,11] and an increase of its unit cell volume [8,12]. These two phenomena provide an explanation of the swelling observed in other irradiated ceramics [27,31-33]. Thus, we decided to compare the measured step with the damaged thickness (Figure 1) to estimate the volume swelling. The results show that  $\text{Ti}_3\text{SiC}_2$  swells by  $2.2 \pm 0.8 \%$  for an irradiation dose of 4.3 dpa.

To determine whether such an estimation of the swelling induced by ion irradiation is relevant or not, we used the same methodology with a polycrystalline  $\alpha$ -SiC sample irradiated in the same conditions as  $\text{Ti}_3\text{SiC}_2$  (inducing an average of 3.1 dpa over 800 nm). We estimated a swelling of  $16.4 \pm 1.3 \%$  (step of  $131 \pm 10 \text{ nm}$ ), which is in agreement with the literature. Actually, due to its amorphization for doses higher than 0.5 dpa, the swelling of SiC would range between 10

and 20 % [29,34]. Thus, this work shows that our methodology is accurate, and that  $Ti_3SiC_2$  swells less than SiC for irradiations carried out at room temperature.

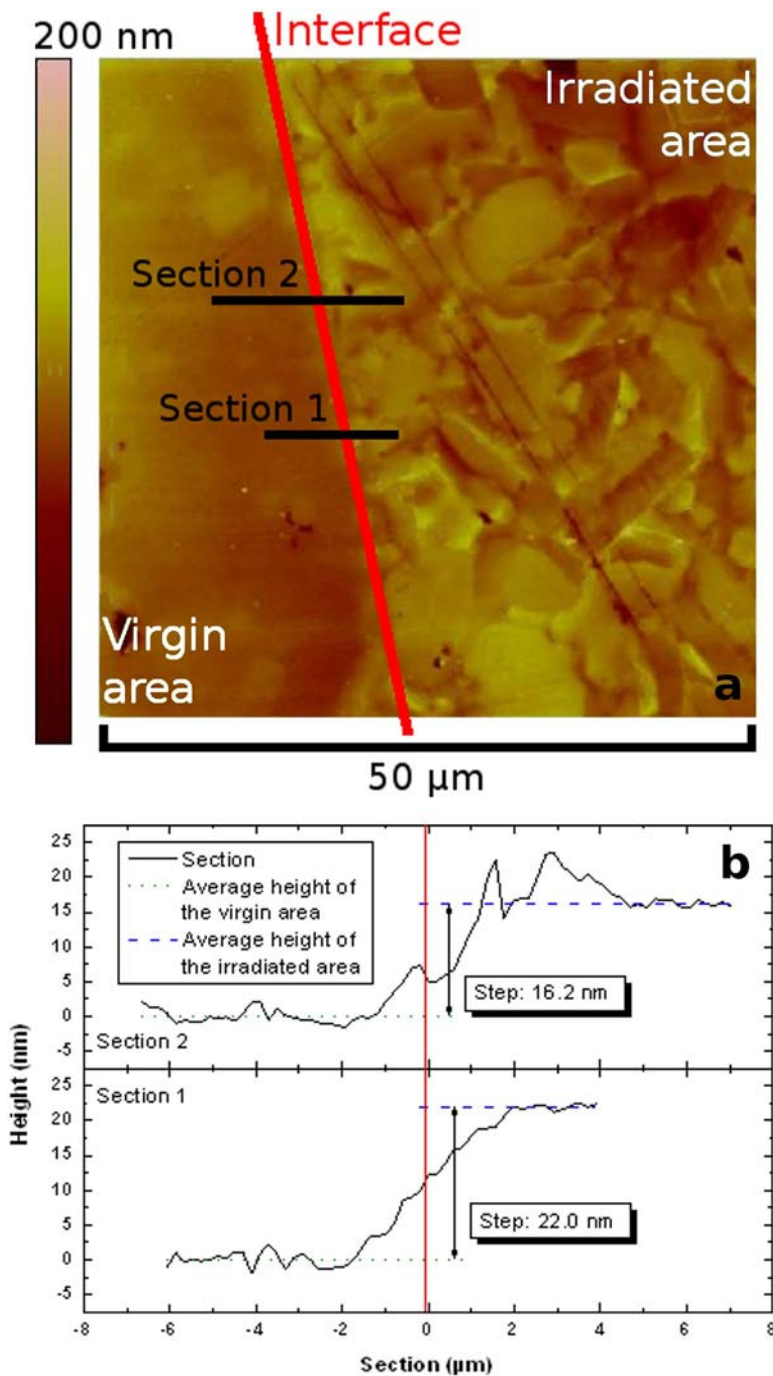


Figure 6: Swelling induced by room temperature irradiation to  $10^{19} \text{ m}^{-2}$ : (a) AFM micrograph of the interface between the irradiated and the virgin areas, and (b) profiles of the sections indicated in (a).

It was not possible to obtain micrographs similar to that of Figure 6 for other irradiation conditions (lower fluence and/or higher temperature). Actually, the height difference between virgin and irradiated areas was not sufficient to be observed by optical microscopy. Therefore, it was not possible to position the AFM tip at the interface between virgin and irradiated areas. Nevertheless, observations of the irradiated areas were carried out for other irradiation conditions. Figure 7 presents typical microstructures of irradiated samples as a function of both the fluence (or number of dpa) and temperature. The dark areas on the micrograph recorded on a sample irradiated at room temperature to  $10^{17} \text{ m}^{-2}$  are due to porosity. This porosity was also visible before irradiation for this sample. They are certainly due to the pull out of surface grains during the sample preparation. In this Figure, one can also see that the revealing of microstructure



appears at room temperature between  $10^{17}$  and  $10^{18} \text{ m}^{-2}$ . When the irradiation temperature is raised to 773 K, it appears between  $10^{18}$  and  $10^{19} \text{ m}^{-2}$ . Eventually, no microstructure revealing is noticeable at 1223 K for a fluence of  $10^{19} \text{ m}^{-2}$ . Therefore, the formation of this microstructure is enhanced when the fluence is increased, or when the temperature is decreased.

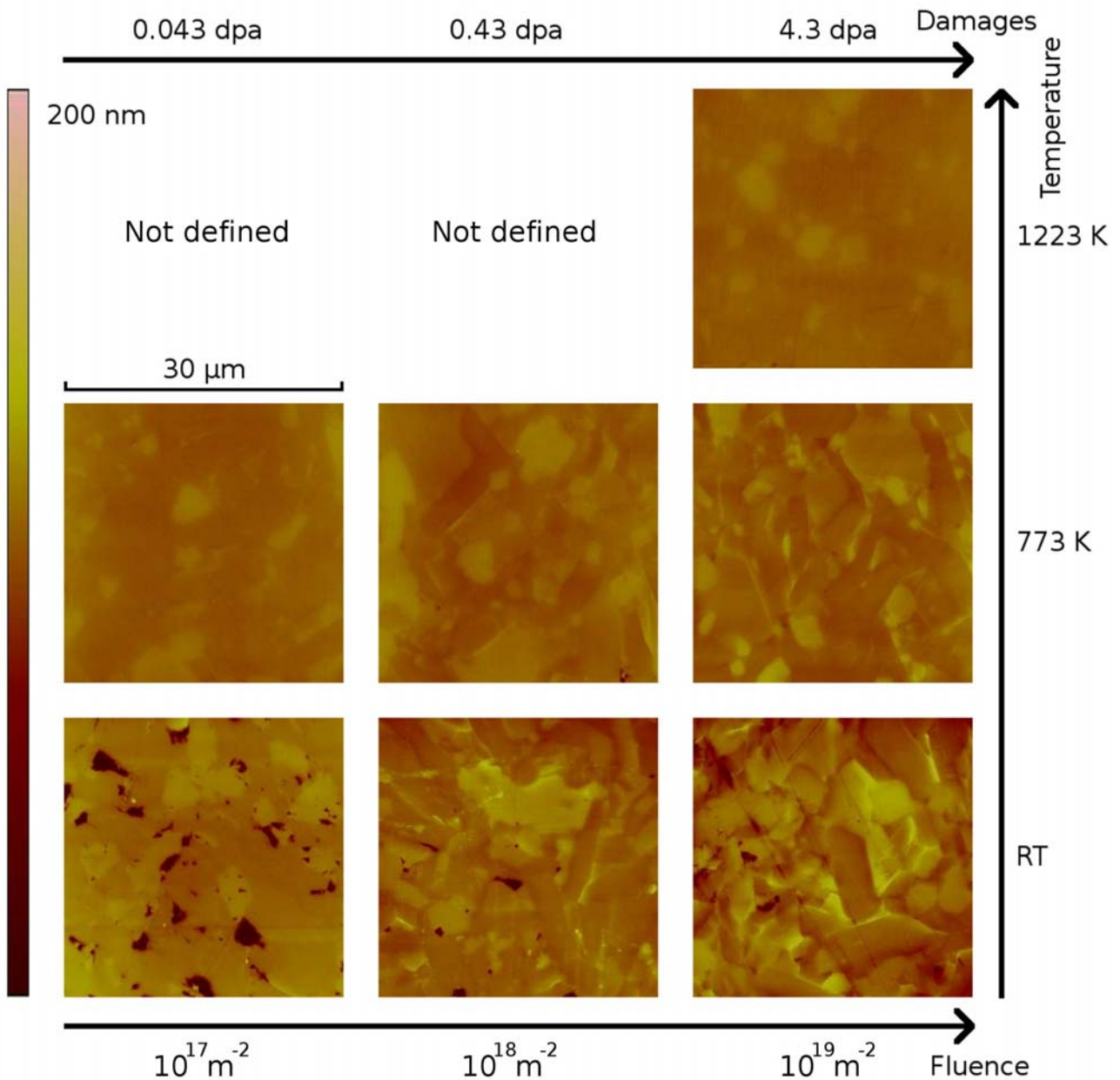


Figure 7: Evolution of the microstructure of irradiated areas as a function of both the temperature and the fluence; RT stands for “room temperature”.

### 3.3. Origin of the microstructure revealing

In a previous paper, we attributed the modification of the microstructure induced by irradiation to an effect of sputtering [15]. However, the sputtering yield does not depend on the irradiation temperature, whereas the present results have shown that the microstructure revealing does depend on it. Therefore, invoking sputtering as a possible cause for the microstructure revealing was a misinterpretation of previous data.

The swelling measurements suggest a new and more relevant explanation of this phenomenon. Actually, upon irradiation at room temperature to  $10^{19} \text{ m}^{-2}$ ,  $\text{Ti}_3\text{SiC}_2$  slightly swells. However, Figure 6 indicates that this swelling is not

the same for all crystallites, inducing a large difference between the measured heights. So, it is likely that anisotropic swelling occurs, owing to the hexagonal close-packed structure of  $\text{Ti}_3\text{SiC}_2$ . Such an anisotropic swelling has already been observed in other materials presenting anisotropic structures [31,35,36]. Moreover, it has been shown that anisotropic swelling leads to the occurrence of significant stresses in the irradiated area, inducing fractures or microcracks at grain boundaries in polycrystalline materials [31,36]. Therefore, the anisotropic swelling would also explain what we previously thought to be an erosion phenomenon of the  $\text{Ti}_3\text{SiC}_2$  grain boundaries [15]: as shown in Figure 8, microcracks formed in the the grain boundaries of  $\text{Ti}_3\text{SiC}_2$  irradiated at room temperature to  $10^{19} \text{ m}^{-2}$ .

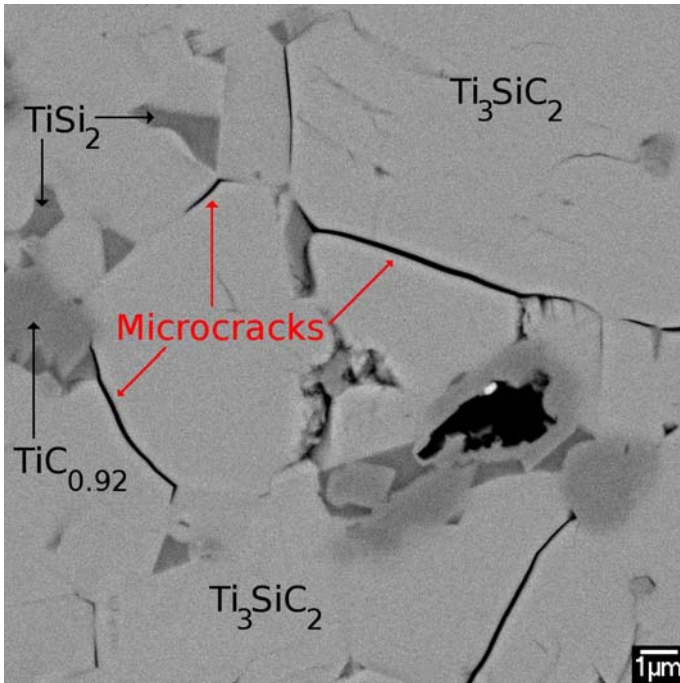


Figure 8: Back-scattered electrons micrograph of the microcrack formation on a sample irradiated at room temperature to  $10^{19} \text{ m}^{-2}$ .

In this work, where other irradiation conditions were explored, microcrack formation was not observed in other samples. This result confirms the hypothesis that both the formation of microcracks and the microstructure revealing are due to anisotropic swelling of  $\text{Ti}_3\text{SiC}_2$ , and that, in the conditions investigated, the swelling of  $\text{Ti}_3\text{SiC}_2$  decreases with decreasing ion fluence and/or with increasing irradiation temperature.

### 3.4. $\text{Ti}_3\text{SiC}_2$ swelling model

Several authors have discriminated different regimes of swelling in ceramics, which depend on the irradiation temperature [31,32]. At low temperatures, irradiation creates point defects or defect clusters, which lead to the amorphization of the material at high fluence. Defect creation induces a swelling, which increases with increasing fluence, and saturates when amorphization is completed (“amorphization regime”). Above the “critical amorphization temperature  $T_c$ ”, temperature at which the damage recovery rate is equal to the damage rate, amorphization of the material does not occur, even at very high fluences. The value of  $T_c$  varies as a function of the nature of the material: for instance, 200 K for graphite [37], 200-250 K for  $\text{Al}_2\text{O}_3$  [27], 400-650 K for SiC [38,39]. Thus, above  $T_c$ , the swelling increases with increasing fluence up to saturation, but it is only due to defect creation (“saturatable regime”). As the material does not become amorphous, the saturation swelling is much smaller than that measured in the amorphization regime, and it steadily decreases with increasing temperature. This decrease is generally attributed to the recombination of Frenkel pairs created in the collision cascades, which is enhanced at high temperature. Eventually, for temperatures high enough to allow vacancies to be significantly mobile, vacancy clusters can form and grow in cavities (or voids). In this “non-saturatable regime”, extended defects are the major cause of swelling, which becomes fluence dependent, and increases with increasing temperature. If  $\text{Ti}_3\text{SiC}_2$  follows this model of swelling, it should be in the saturatable regime, whatever the irradiation temperature is. This assumption stems from the following statements:

- (i) Nuclear collisions create defects in  $\text{Ti}_3\text{SiC}_2$  without leading to amorphization [11,12,15], even for the highest studied dose (4.3 dpa), whereas SiC, which is in the amorphization regime at room temperature, becomes amorphous above 0.5 dpa. This suggests that  $\text{Ti}_3\text{SiC}_2$  is not in the amorphization swelling regime;
- (ii) No extended defects have been observed by transmission electron microscopy analyzes [8], even for the highest temperature (1223 K), suggesting that  $\text{Ti}_3\text{SiC}_2$  is not in the non-saturatable regime;
- (iii) The swelling decreases and increases respectively with the temperature and the dose (see Section 3.2.). This behavior is typical of the saturatable regime.

Therefore, the critical amorphization temperature of  $\text{Ti}_3\text{SiC}_2$  would certainly be lower than room temperature, and the transition temperature between the saturatable and the non-saturatable regimes would be higher than 1223 K.

However, as  $\text{Ti}_3\text{SiC}_2$  possesses some properties generally attributed to metals, and has also a behavior under electronic excitations similar to that of metals [8], it could follow another model. According to the literature, metals do not seem to become amorphous for very high dose [40,41], in the temperature range studied in this work. Moreover, the swelling of metals does not saturate, but increases linearly with the dose due to the agglomeration of point defects into extended defects, whatever the irradiation temperature. Eventually, a temperature increase usually induces an increase of the swelling up to a maximum for a critical temperature above which the swelling decreases [42]. Thus, considering that such a critical temperature is ever reached for irradiations at room temperature, the results presented in this work also match with this model, and complementary irradiations, such as irradiations at lower and higher temperatures, or creating more damage, are needed to better understand the swelling behavior of  $\text{Ti}_3\text{SiC}_2$ .

## 5. Conclusion

The first aim of this study was to confirm that the microstructure revealing, observed on  $\text{Ti}_3\text{SiC}_2$  irradiated at room temperature with 4 MeV Au ions to  $10^{19} \text{ m}^{-2}$ , depends on the crystallite orientation. This result was confirmed by combining AFM and EBSD observations that show similarities in the shape and size of both the revealed grains and the crystallites of the samples. Moreover, EBSD analyzes allowed the highlighting a hitherto unexpected result: the secondary phase  $\text{TiSi}_2$ , present in the studied specimen, is highly textured. We conjecture that the formation of a liquid phase during the sample preparation could be the cause of this strong texturing.

The second goal of this work was to develop a methodology to estimate by AFM the volume swelling induced by ion irradiation. The method was validated with measurements performed on an irradiated SiC sample that match the result found in the literature. Using this method, we showed that  $\text{Ti}_3\text{SiC}_2$  weakly swells at room temperature ( $2.2 \pm 0.8 \%$ ) for an average irradiation dose of 4.3 dpa, whereas the swelling of SiC irradiated in the same conditions reaches 16.4 %. Furthermore, we showed that, in the temperature and damage range of our study, the higher the temperature or the lower the amount of damage, the lower the swelling of  $\text{Ti}_3\text{SiC}_2$ . However, complementary irradiations are needed to determine the swelling model applicable to  $\text{Ti}_3\text{SiC}_2$ , which can be either that implemented for ceramics or that implemented for metals.

Finally, by comparing the micrographs obtained by using both AFM and FEG-SEM, we showed that the microstructure revealing induced by irradiation is due to an anisotropic swelling of  $\text{Ti}_3\text{SiC}_2$ . Nevertheless, since neither microstructure revealing, nor crack formation have been observed on the sample irradiated at 1223 K to  $10^{19} \text{ m}^{-2}$ , we can conclude that, from the swelling point of view,  $\text{Ti}_3\text{SiC}_2$  seems to present interesting prospects for use as a cladd component of GFR.

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